

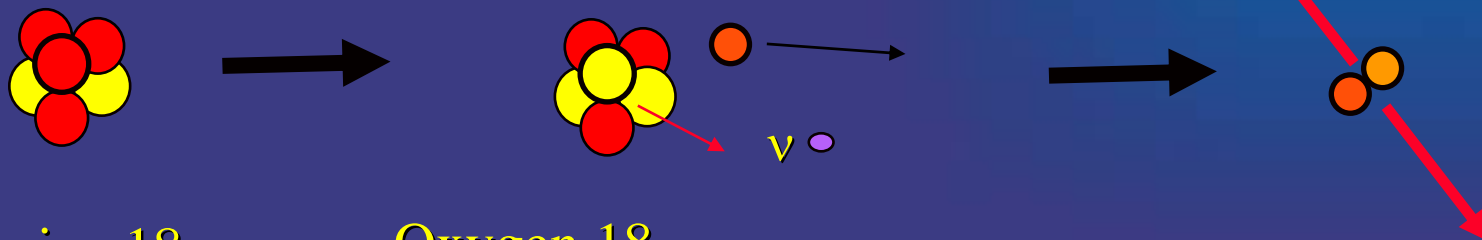


Hospital Cyclotrons: Radiation Safety Aspects

**Matthew
Griffiths**

Isotope Production.

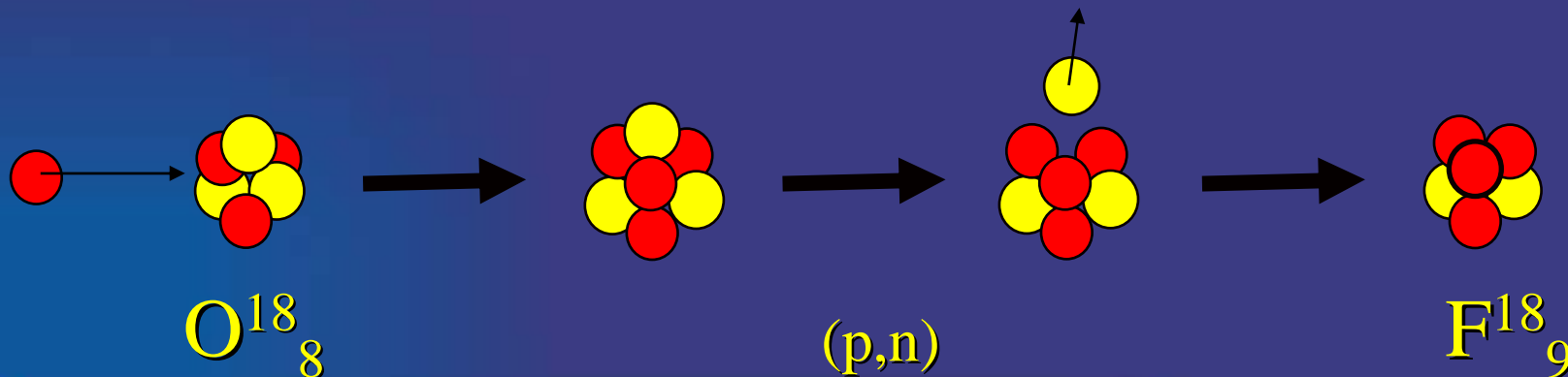
Positron decay is a way for an atom with too many protons to get to a more relaxed state.



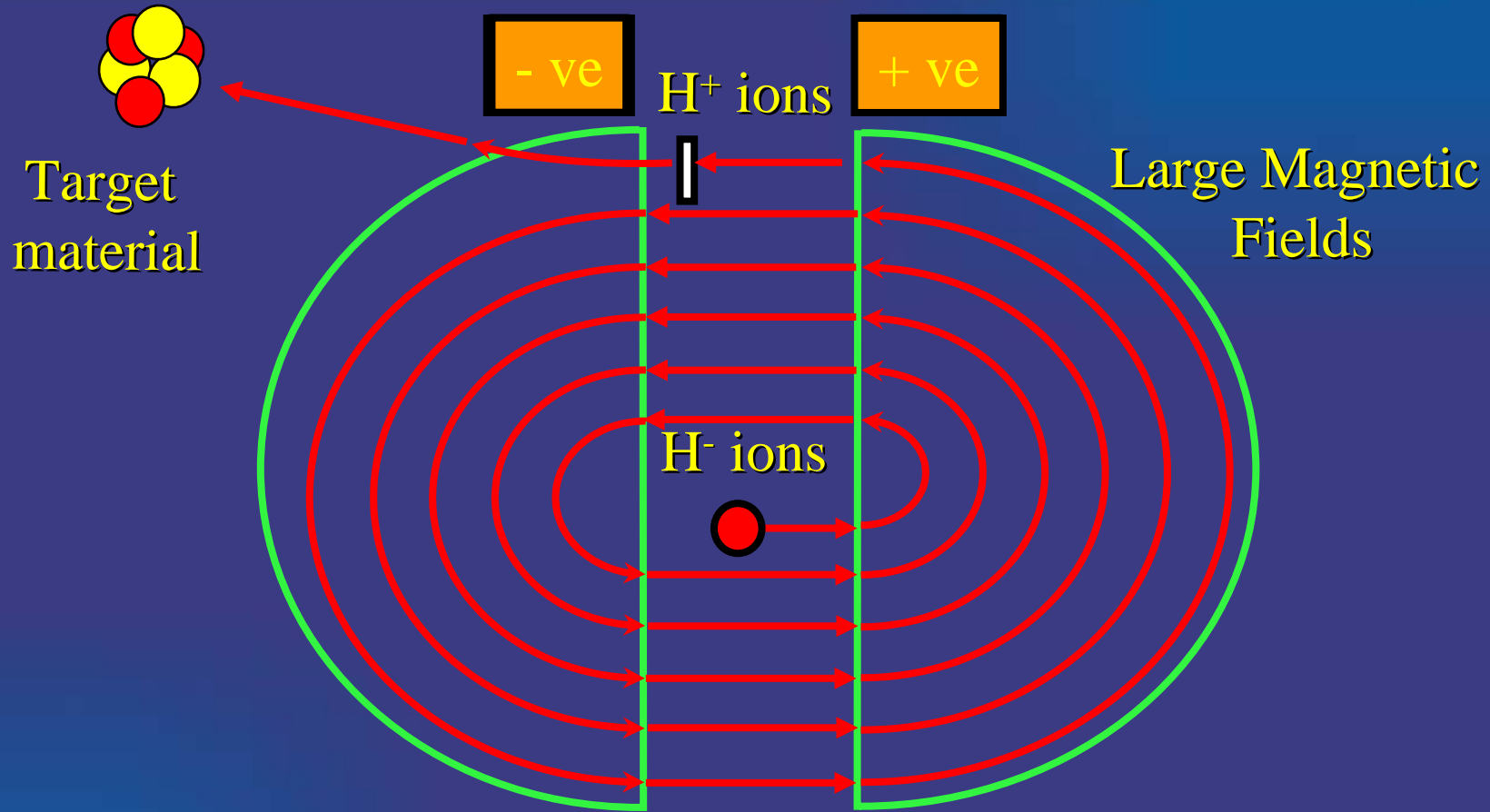
Fluorine 18
excess Proton

Oxygen 18
Neutron, Neutrino,
Positron and Gamma rays

Production: Take a stable target atom and add a proton.



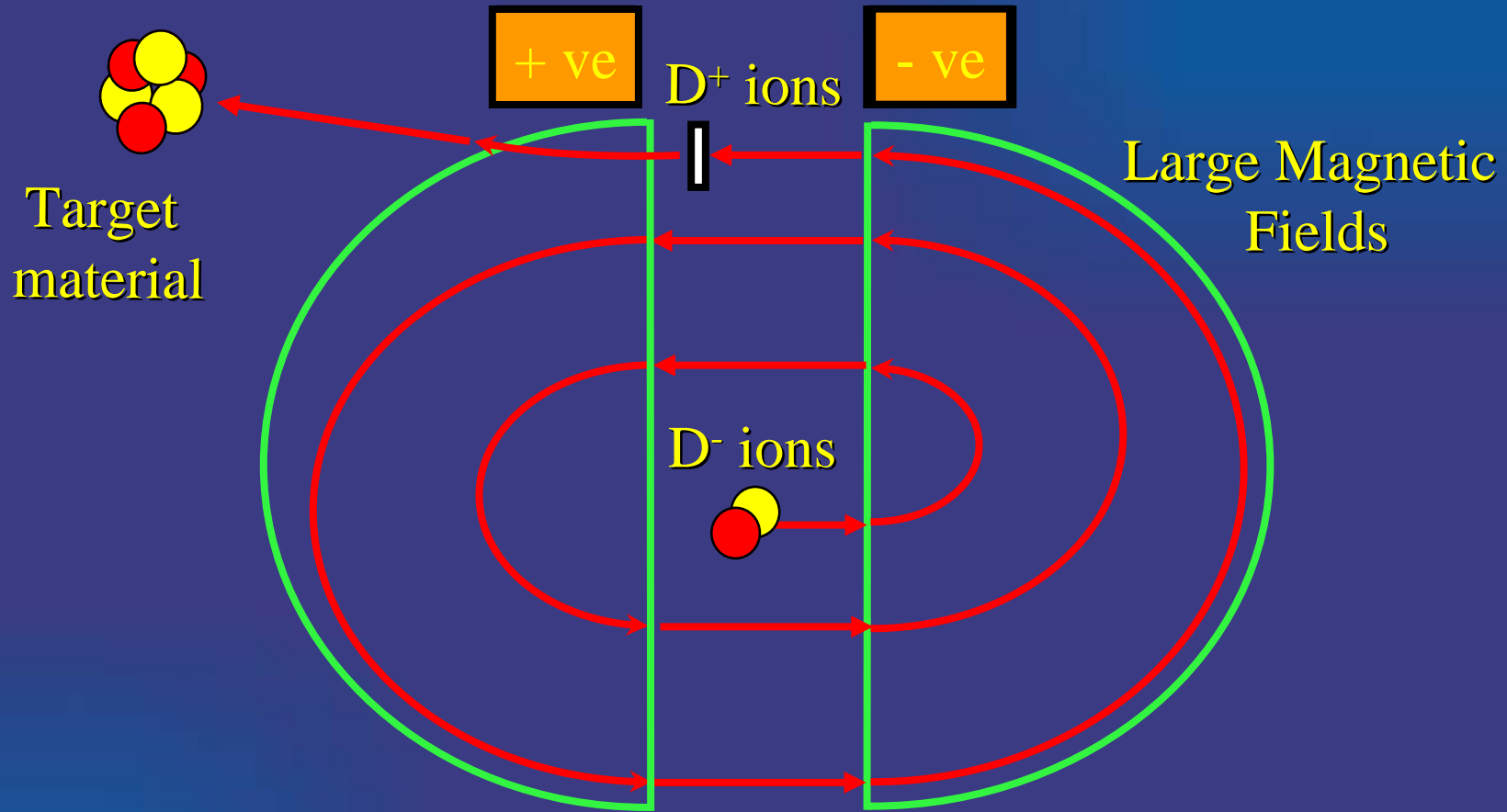
Cyclotron



Electric potentials accelerate ions.
Magnetic fields cause ions to travel in circles.



Cyclotron



Electric potentials accelerate ions.
Magnetic fields cause ions to travel in circles.



Cyclotron

Not a Nuclear Reactor.

Needs no radioactive fuel.

Does not produce radioactive waste or emissions.

An electrical device that can be turned off.

(Very big brother to a TV set)

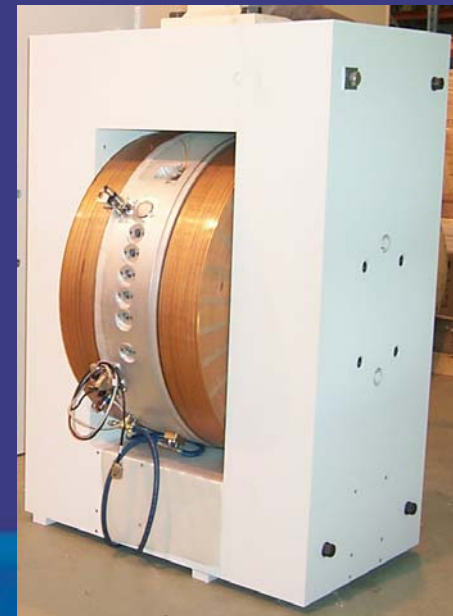
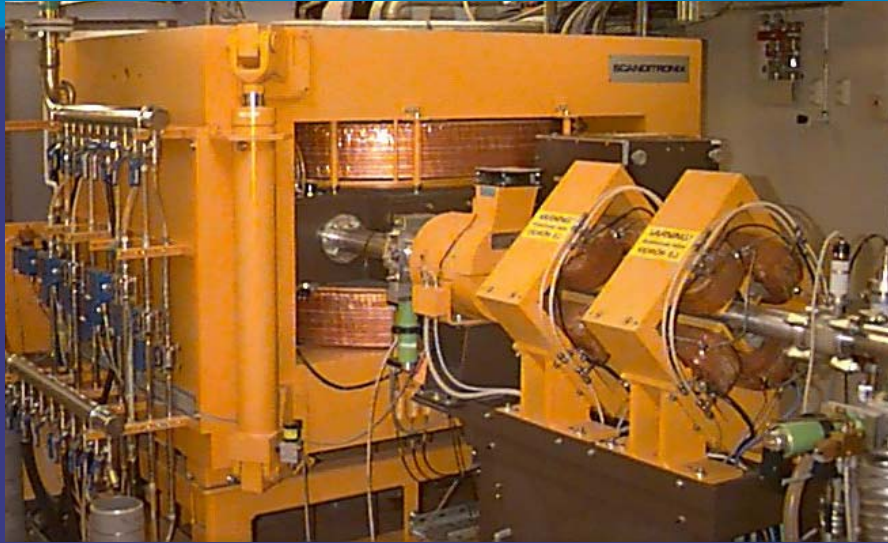
Requires a small amount of a stable material as a target.

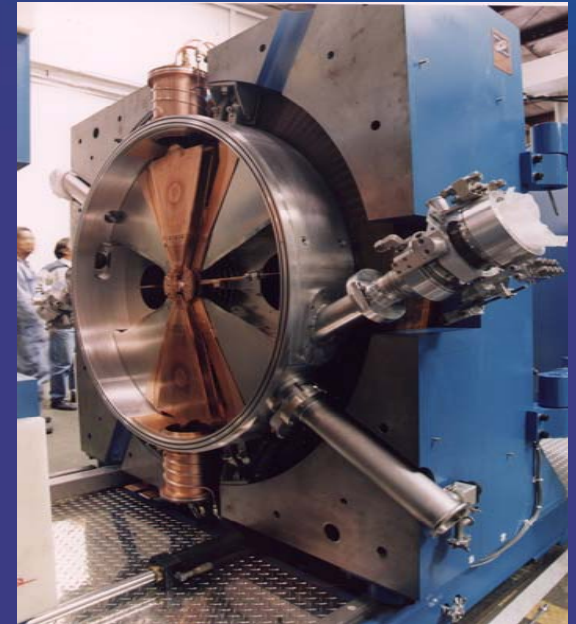
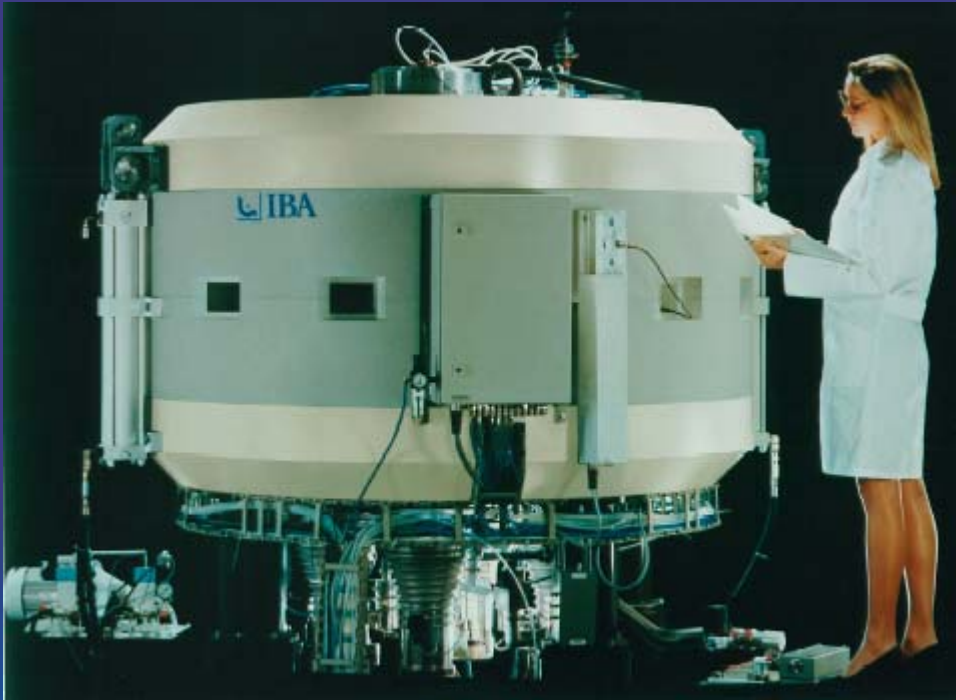
Produces a small volume of a highly specific radioactive product.

Does not become radioactive.



CYCLOTRONS



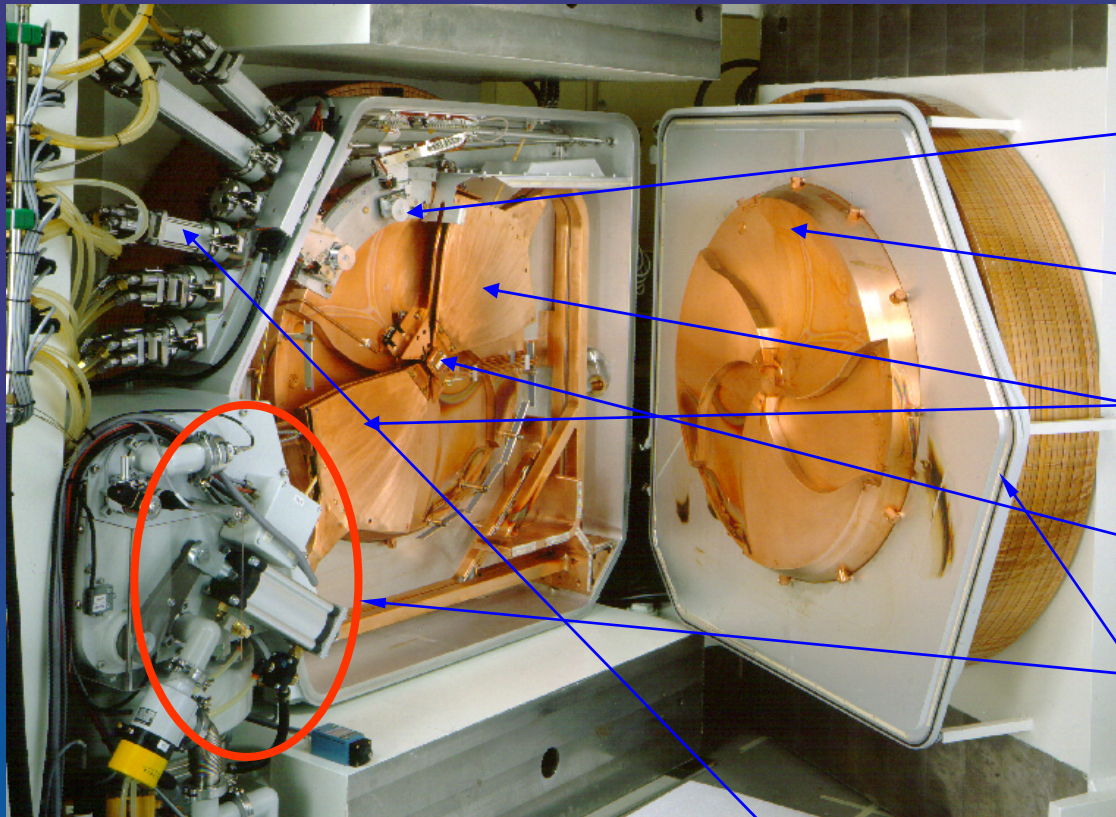


Cyclotron subsystems

- Magnet system
- RF system
- Ion source system
- Extraction system
- Diagnostic system
- Vacuum system
- Control System
- Target system
- Cooling system



Cyclotron subsystems



Extraction system

Magnet pole

RF system (Dee's)

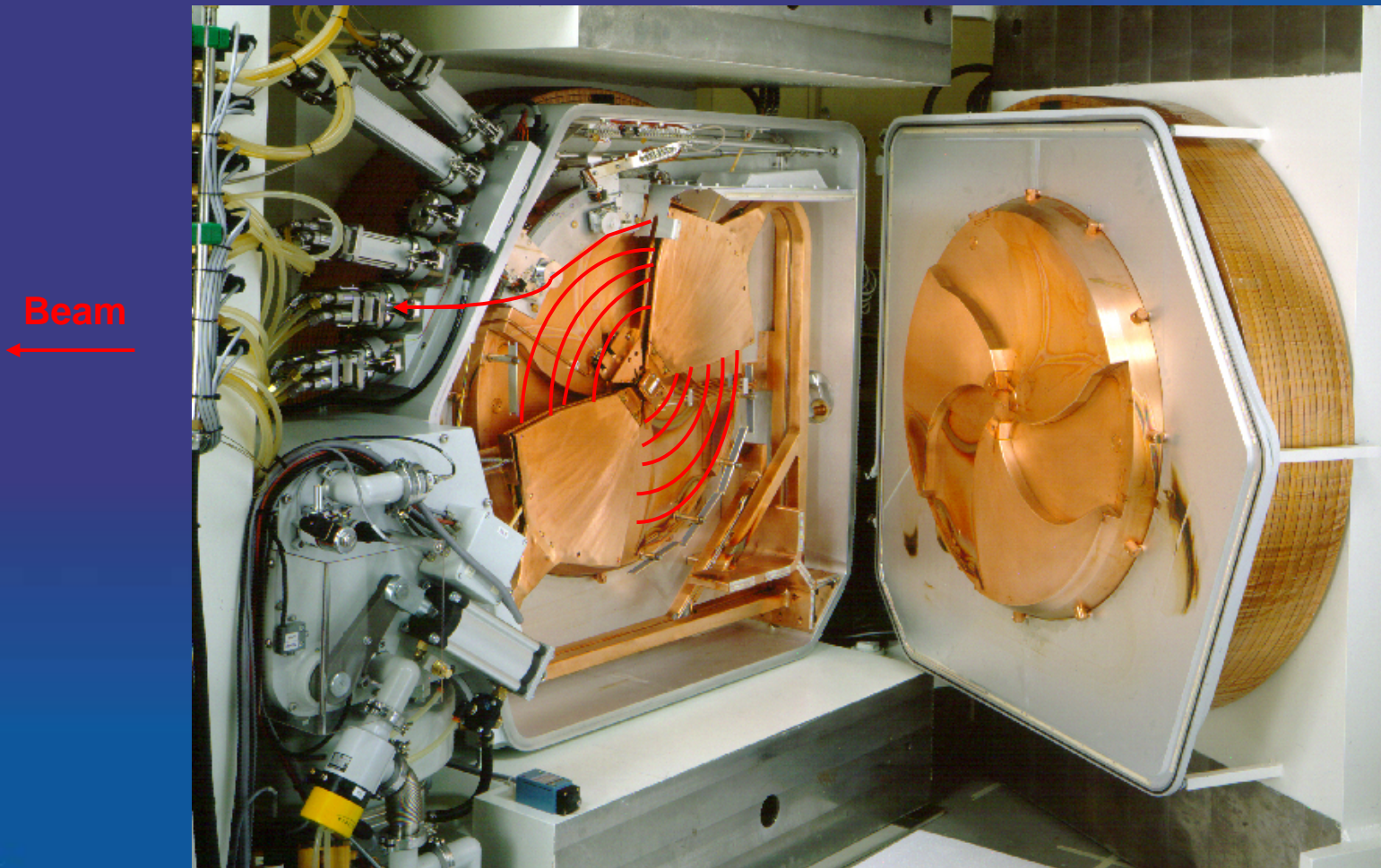
Ion source

Vacuum system

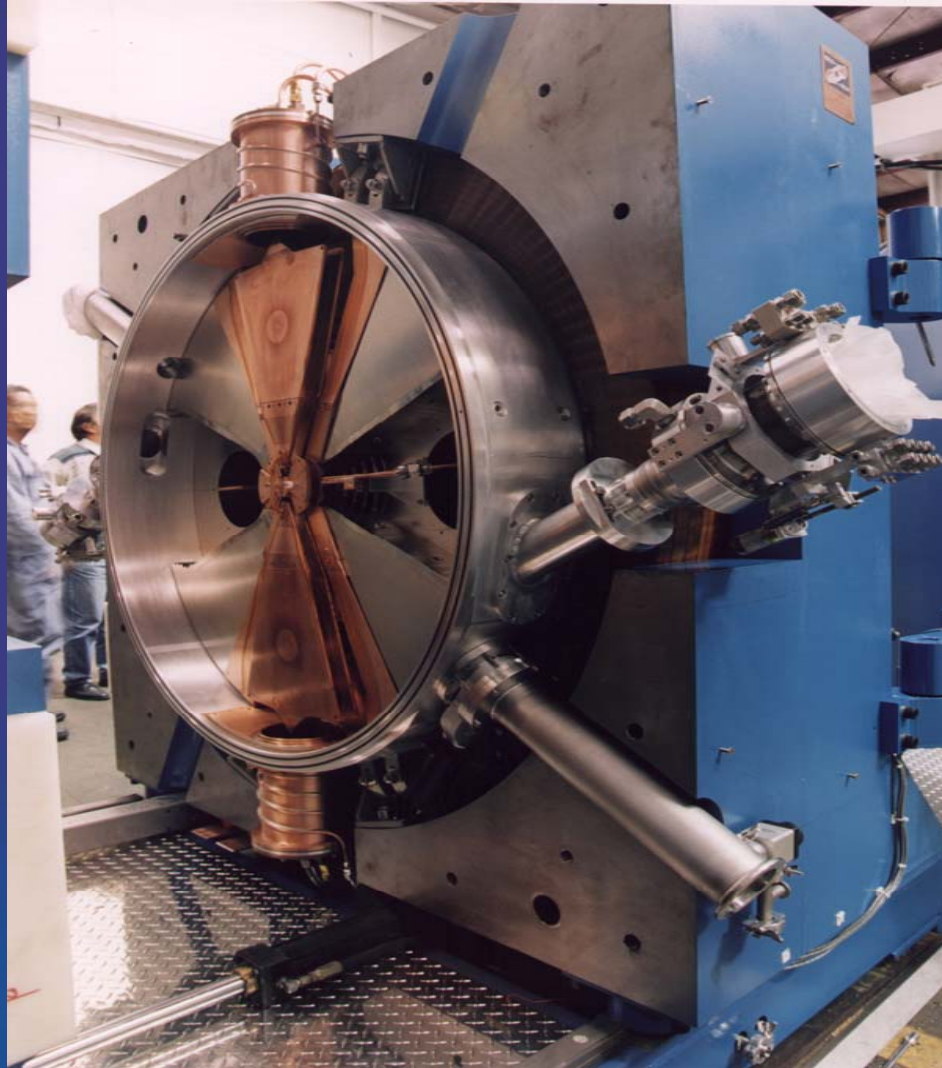
Targets



Beam acceleration







Cyclotron Radiation Source Terms:

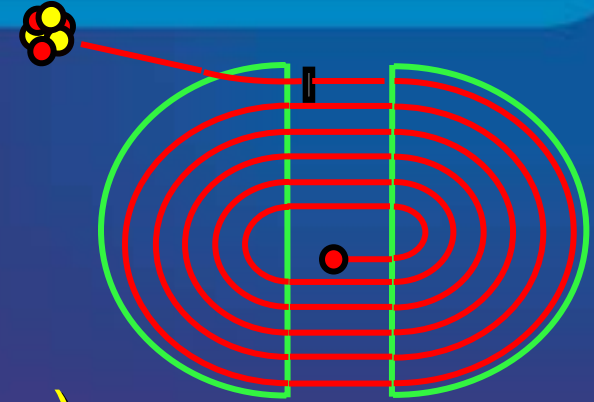
- Proton acceleration

lost protons

(40% due to electron loss)

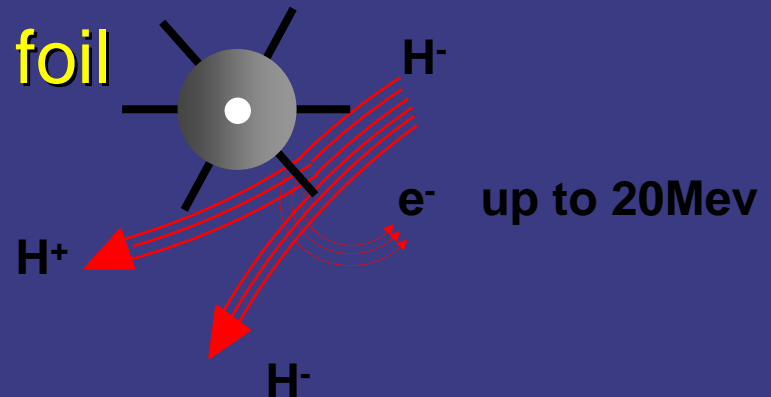
$(p, \underline{x}) + \gamma$ with vacuum chamber

$x =$ neutrons deuterons tritons alphas



electrons after stripper foil
x rays

Carbon (stripper) foils



Cyclotron Radiation Source Terms:

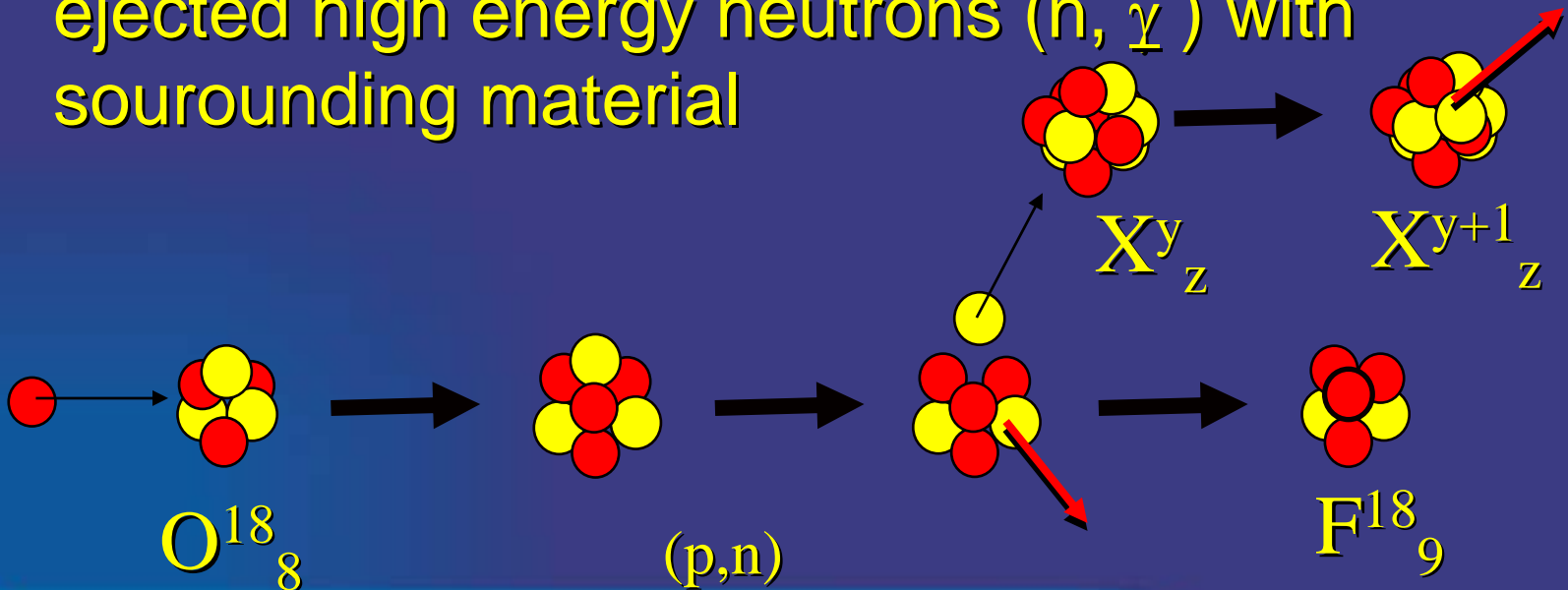
- Bombardment

$(p, \underline{n}) + \underline{\gamma}$ from target material

$(p, \underline{x}) + \underline{\gamma}$ from target body + collimators

$x =$ neutrons deuterons tritons alphas

ejected high energy neutrons $(n, \underline{\gamma})$ with surrounding material



Cyclotron Radiation Source Terms:

<u>Proton attenuation</u>	μA	no./s	MeV
•Lost H- with a range of energies	32	2.0×10^{14}	0-16.5
•Protons leaving the acceleration	48	3.0×10^{14}	16.5
•Tantalum collimator losses	8	5.0×10^{13}	16.5
•Foil $20\mu\text{m}$, He 30mm, Foil $50\mu\text{m}$ Havar (Co 42.5%, Cr 20%, Mn 1.6%, Mo 2%, Ni 13%, W 2.8%, Fe 18.1%)			15
• ^{18}O (5% ^{16}O)	40	2.5×10^{14}	15
•Some will impact on back of target (silver)			

H_2^{18}O target for ^{18}F 40 μA of 15 MeV protons (GE PET trace)



Cyclotron Radiation Source Terms:

• Proton interactions

Material	Proton flux [1/s]	Yields [particles/s]					Neutron energy
		Neutrons	Deutrons	Tritons	Alfa	Gamma	E_{av} [MeV]
Al (p,x)	4.0E+13	2.8E+09	6.3E+07	0.0E+00	2.8E+10	6.1E+10	2.45
Si (p,x)	1.5E+14	2.8E+09	1.0E+09	2.1E+06	2.1E+09	2.3E+11	2.02
Cu (p,x)	1.0E+13	3.6E+09	4.7E+06	5.2E+04	1.1E+09	1.5E+10	1.96
Ta (p,x)	5.0E+13	1.0E+11	3.3E+07	2.4E+06	7.0E+07	2.2E+11	1.38
Havar (p,x)	2.5E+14	2.8E+11	2.4E+09	1.9E+08	4.4E+09	4.4E+11	2.29
H ₂ ¹⁸ O (p,x)	2.5E+14	3.2E+11	3.2E+09	0.0E+00	6.4E+11	2.2E+12	1.93
H ₂ ¹⁶ O (p,x)	2.5E+14	0.0E+00	1.2E+06	0.0E+00	4.2E+10	3.2E+11	
¹⁴ N ₂ (p,x)	2.5E+14	1.1E+10	2.1E+10	0.0E+00	2.6E+11	1.3E+12	3.40
Al dummy	2.5E+14	1.3E+11	8.6E+09	0.0E+00	6.3E+11	2.3E+12	2.92

7.13E+11 total
1.41E+11 forward
5.72E+11 isotropic

H₂¹⁸O target
40 μA, 15 MeV protons
(GE PET trace)



•Gamma source terms

(energy per second)

prompt gamma from (p,x) and (n,γ) reactions)

Material	(p,x) [MeV/s]	(n,g) [MeV/s]
Ta	1.7E+11	
Si	6.5E+11	
Pb		1.0E+10
Ag		6.6E+09
Cu + Others	2.4E+10	6.9E+11
Al	1.5E+11	1.7E+10
Fe + SS		5.8E+11
Concrete		3.5E+12
H ₂ O + Oil		2.0E+09
Havar	6.2E+11	
Target - H ₂ ¹⁸ O	3.8E+12	
Target - H ₂ ¹⁶ O	1.9E+12	
Target - ¹⁴ N ₂	3.9E+12	
Target - Dummy (Al)	6.6E+12	



Net dose rate

- Gamma: Sieverts per hour
- Neutron: tens of Sieverts per hour

QLD design criteria $1 \mu\text{Sv/hr}$



Reducing the radiation to safe levels

- Attenuation
- Minimize + localize the neutron interactions
(n, γ) (n,2n) (n, α)

Two basic choices

- Self shielded cyclotron
- Cyclotron vault



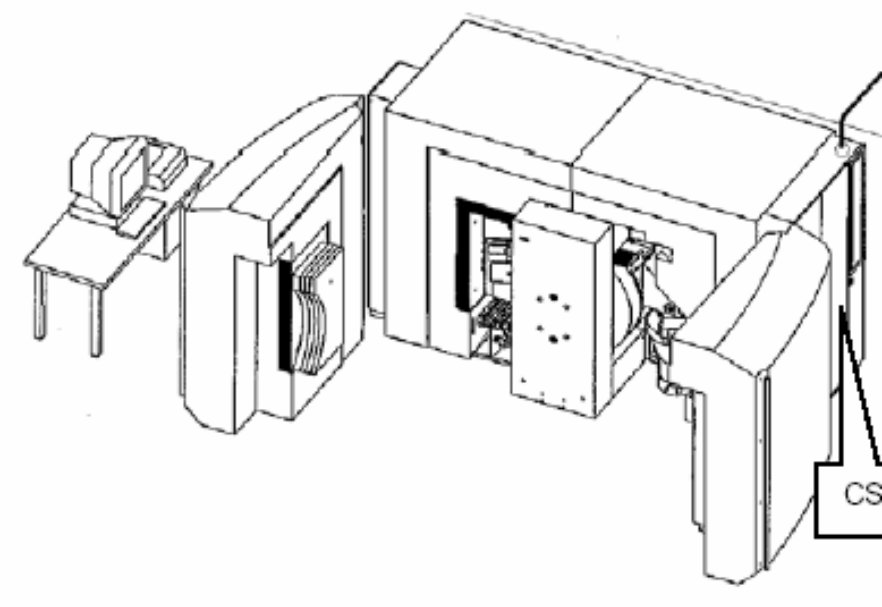
Self Shielded

Shielding encases cyclotron

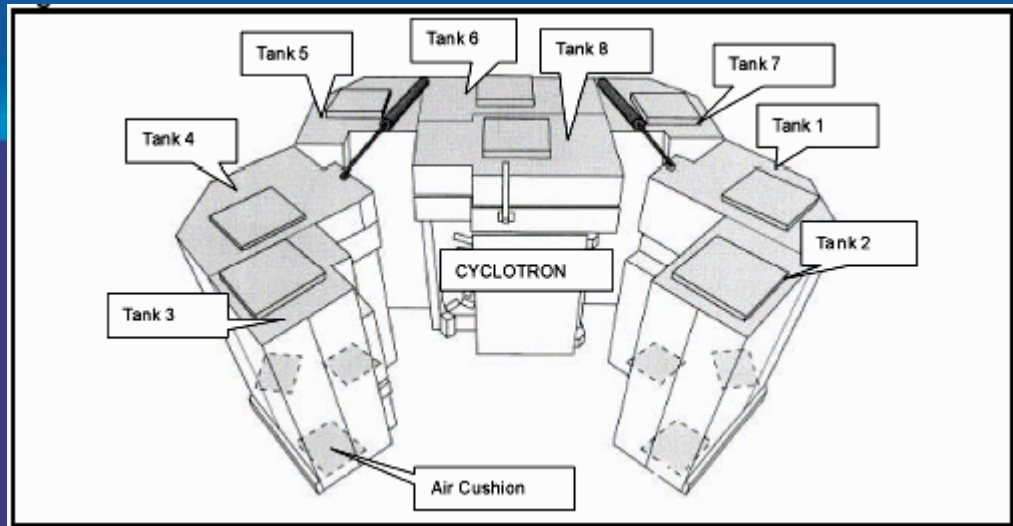
- Moderate the neutrons.
Polyethylene water wax
- Absorb neutrons with min activation and γ rays.
Boron
- Attenuate γ rays
Lead Concrete
- Still may require thick walls 50-300 mm



GE MiniTrace



PETTrace



Polyethene
(3 %B)

Lead shields

Water tanks (3.5% B)

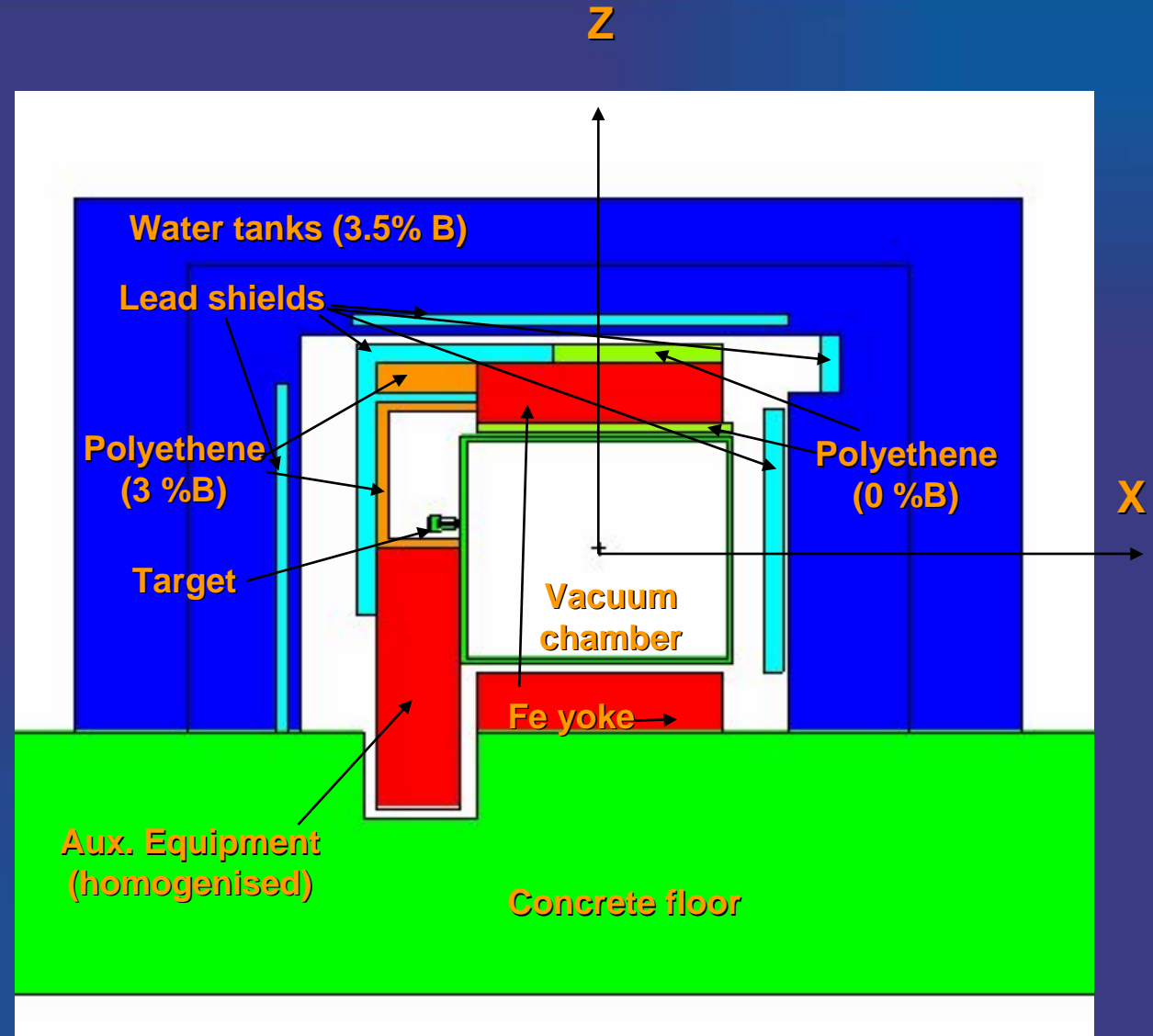
Target

Windings

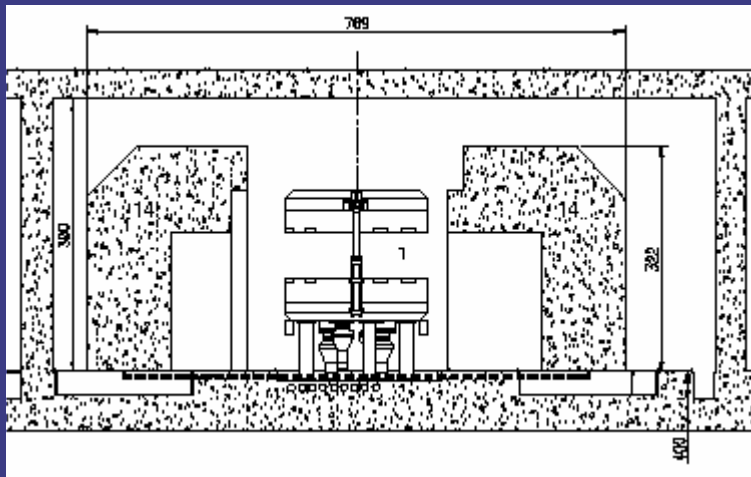
Fe yoke

47 000 kgs

GE PETtrace

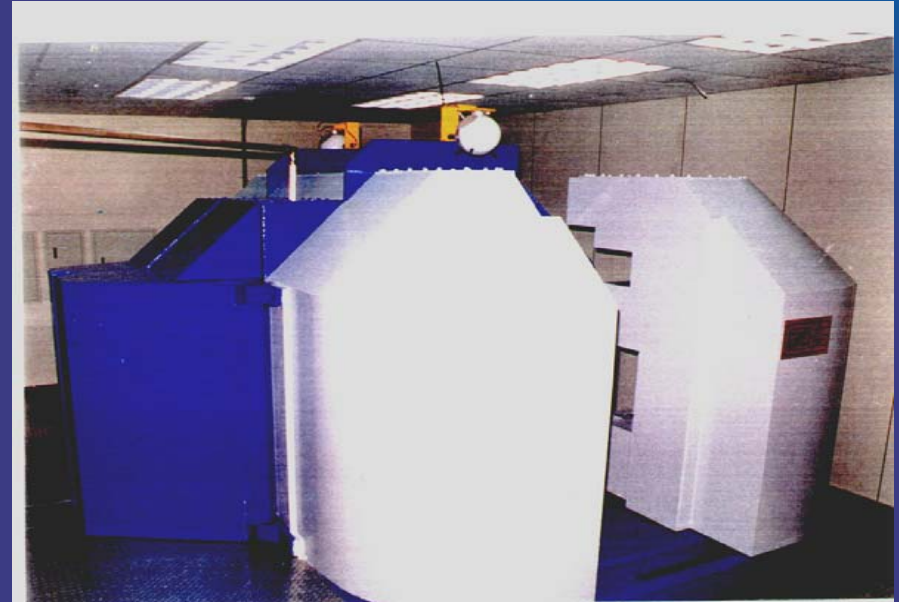


IBA



2 x 45 000 kgs

EBCO



60 000 kgs



Cyclotron Vault

Place cyclotron in shielded enclosure

- Use bulk material to attenuate Neutron and Gamma dose.

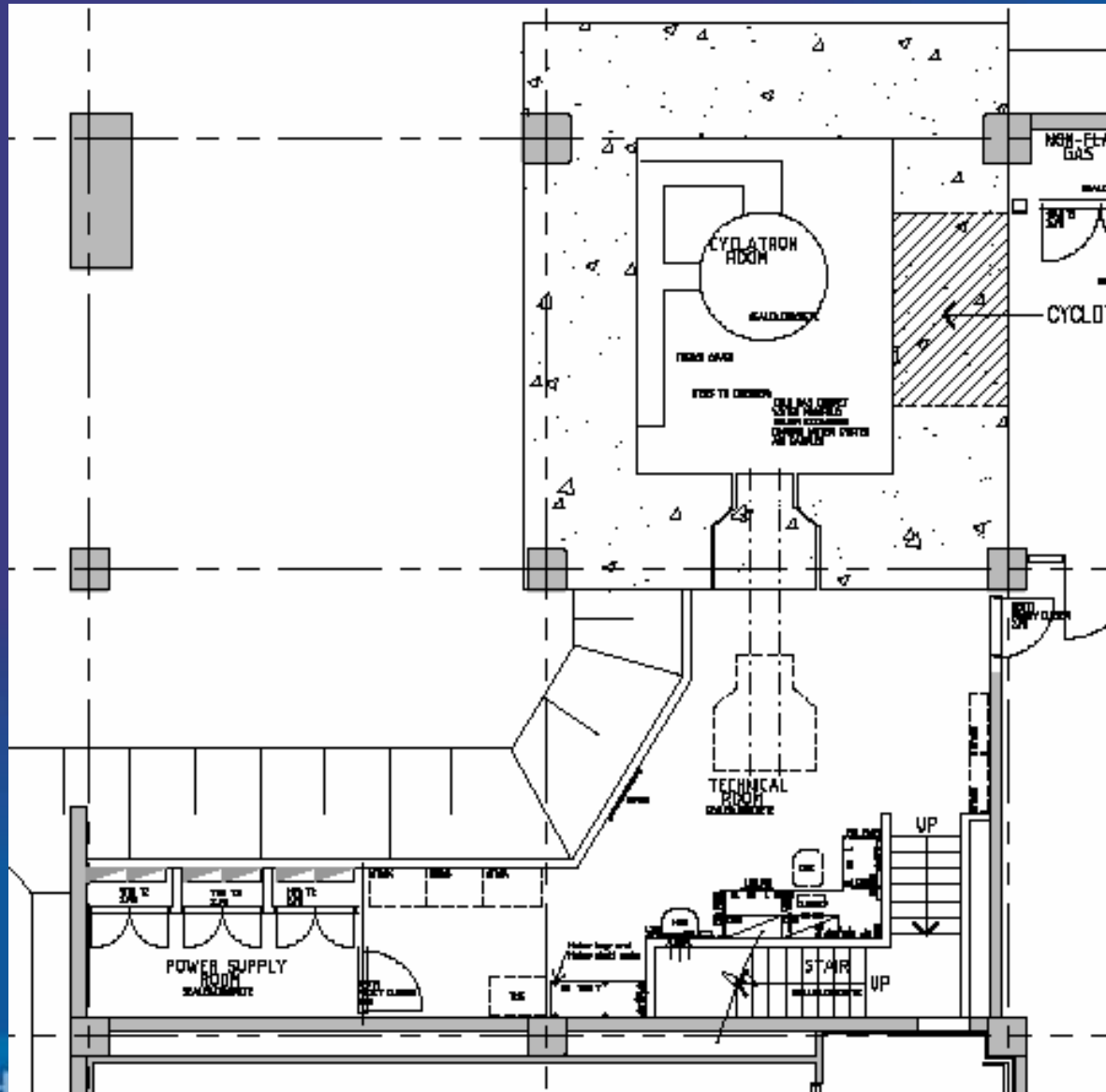
Usually concrete, 1.6 - 2.0 m

Can use additives to increase neutron capture

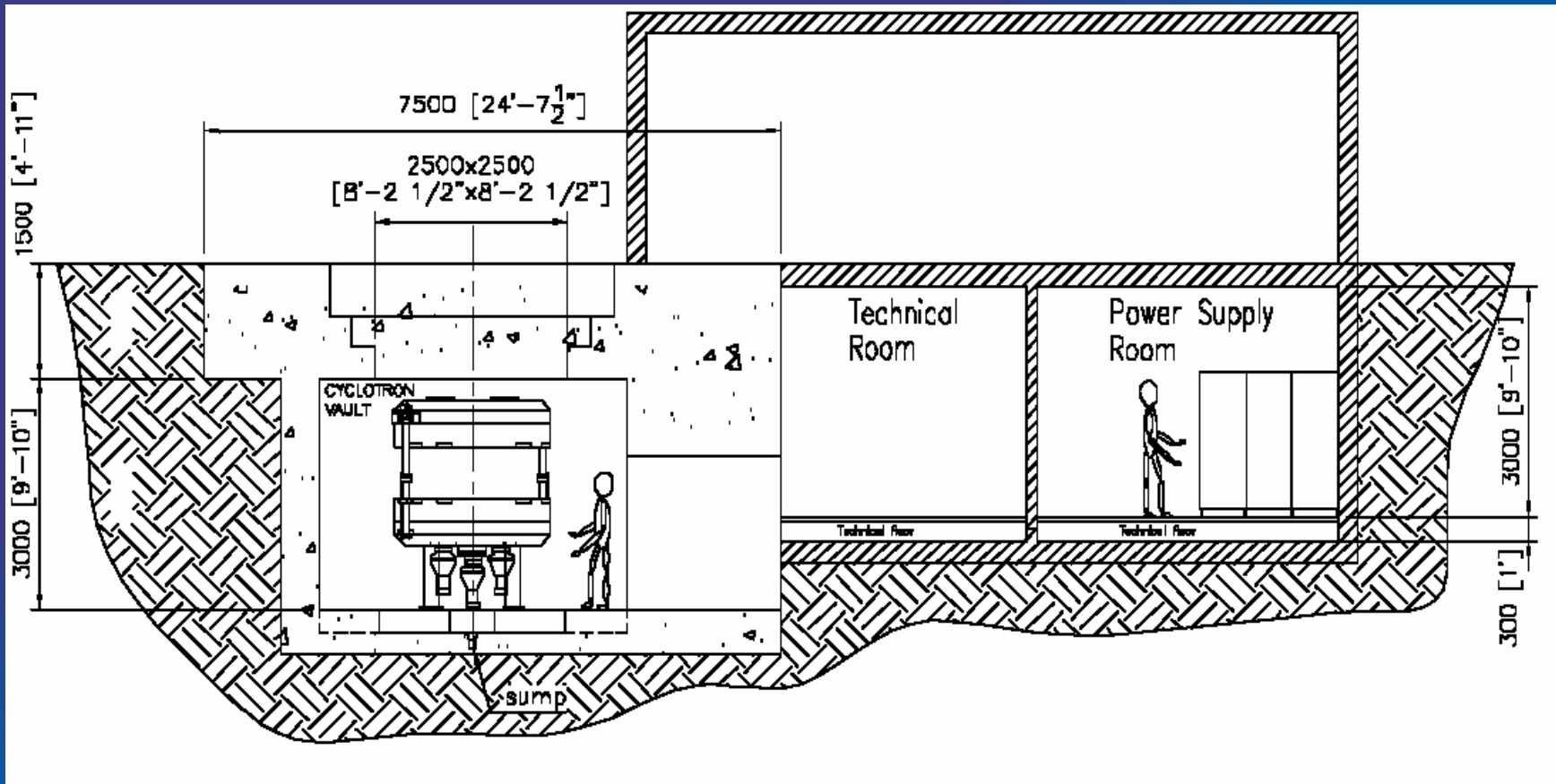
- Access via either a maze or a door (plug)



Vault



Partially underground vault



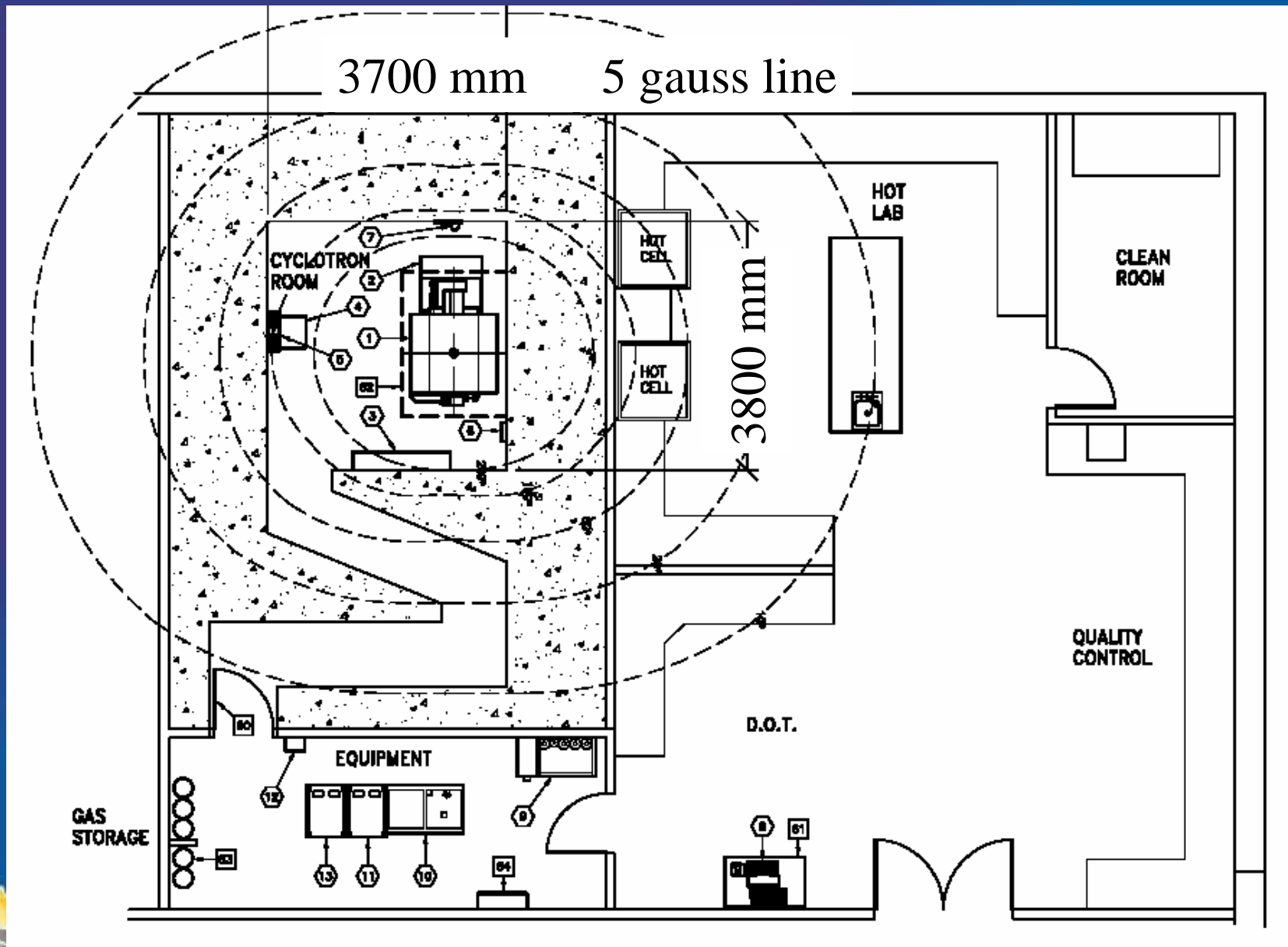
Bunker Doors







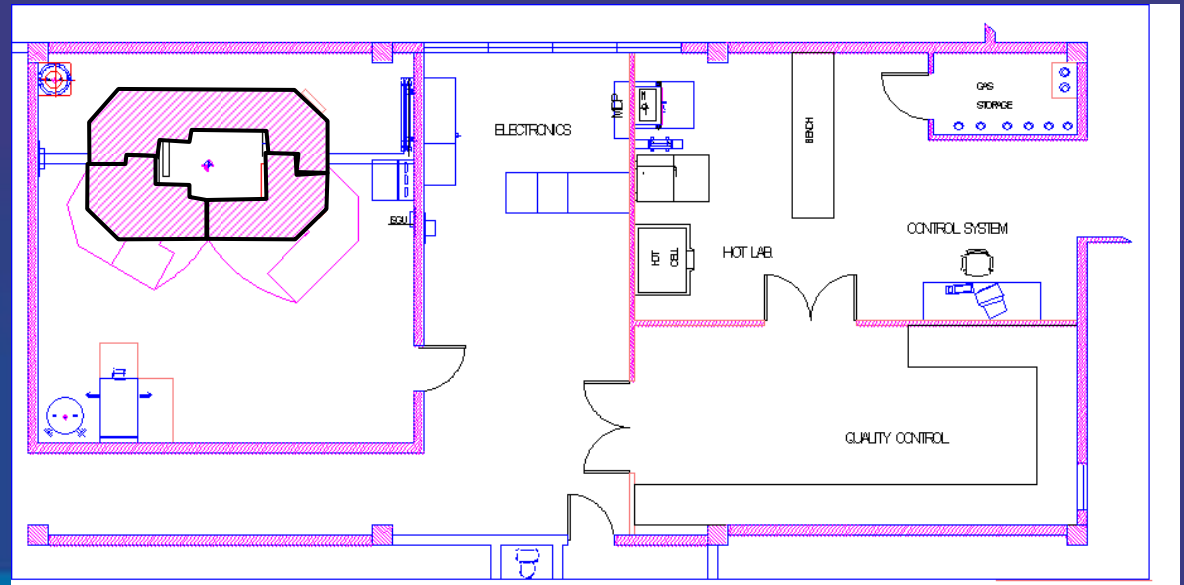
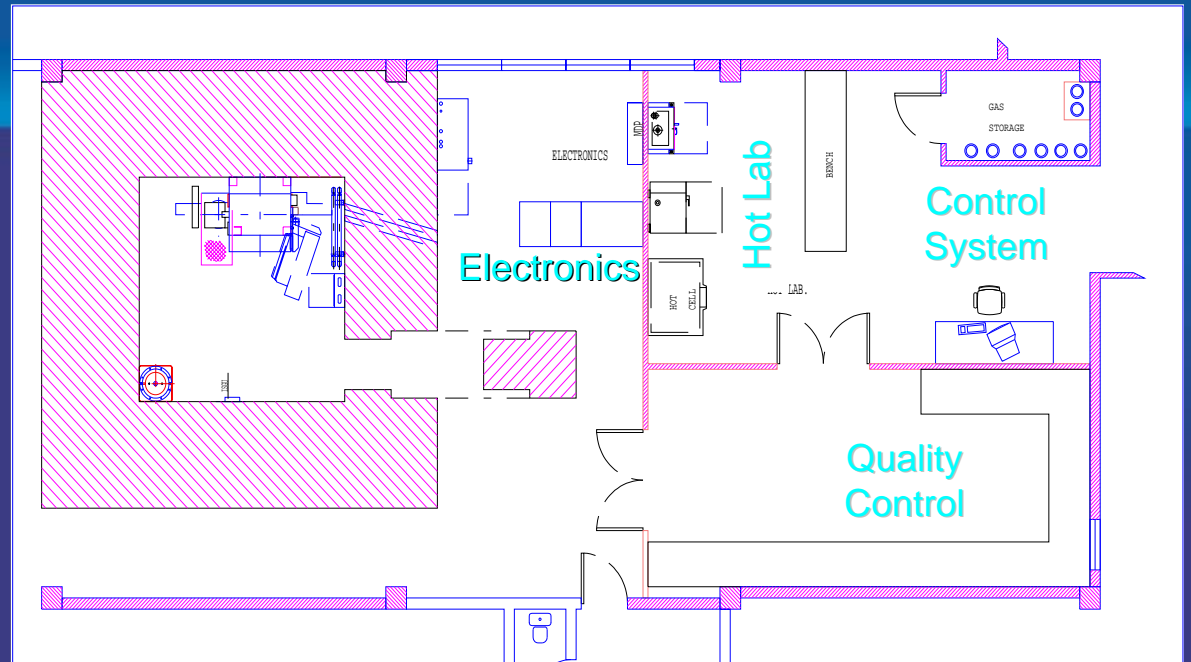
Maze entrance

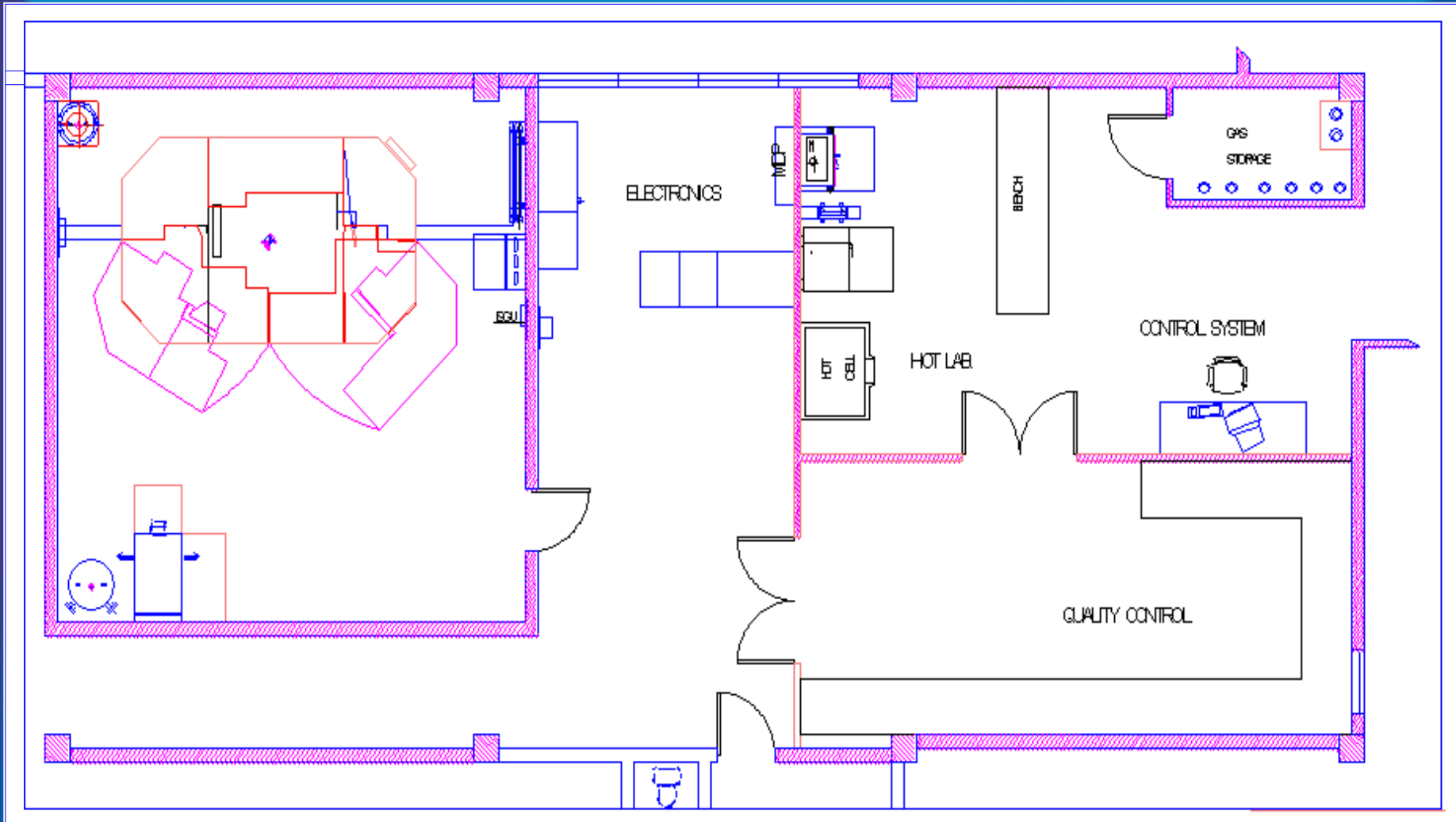


Cyclotron vault

VS

Self shield





Vault vs Self Shield

Vault

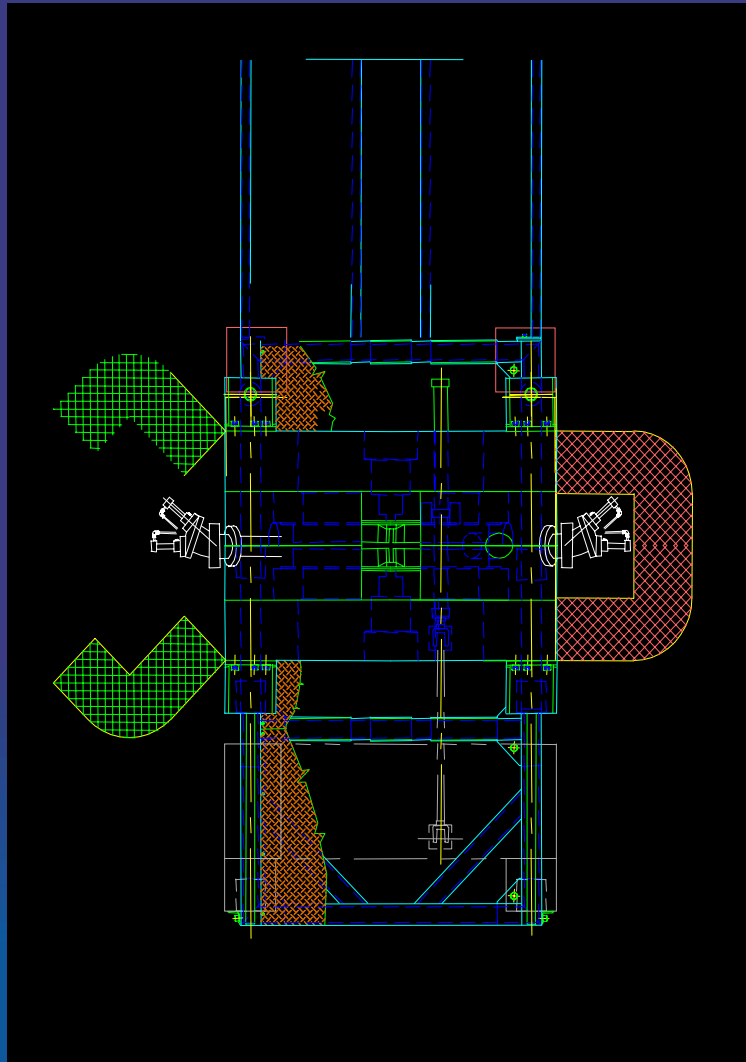
- future growth, beam lines
- maintenance access
- may cost less
- site specific radiation calcs (neutrons)

Self Shield

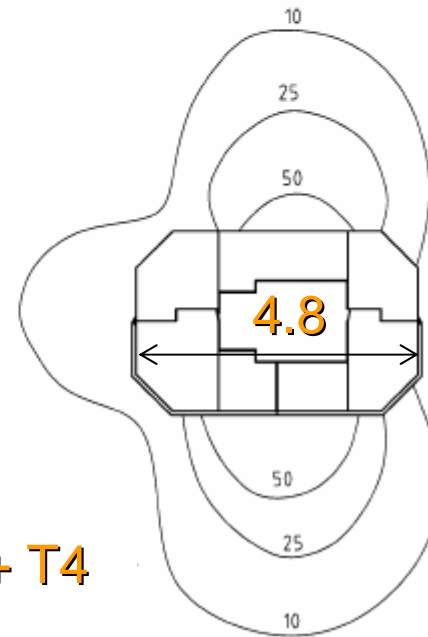
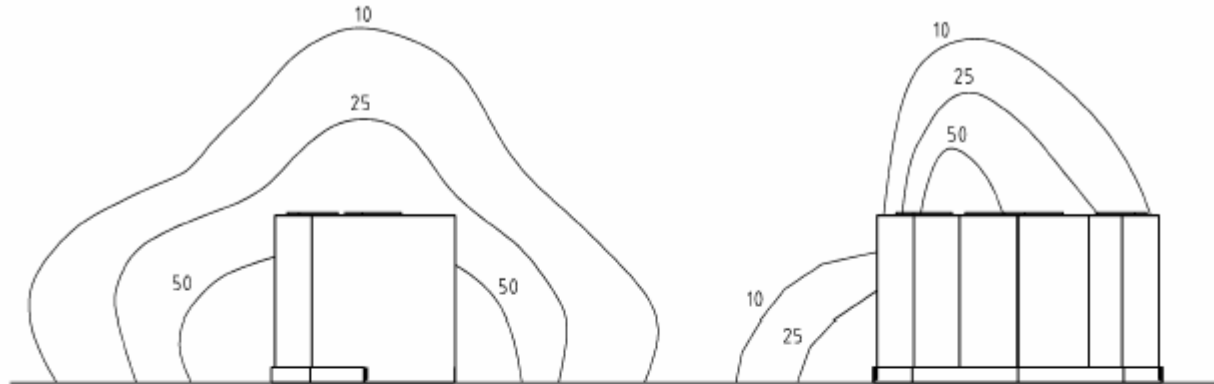
- limited additional targets
- limited maintenance access
- known radiation source
- smaller in height



Limited Local Shielding + Light Vault



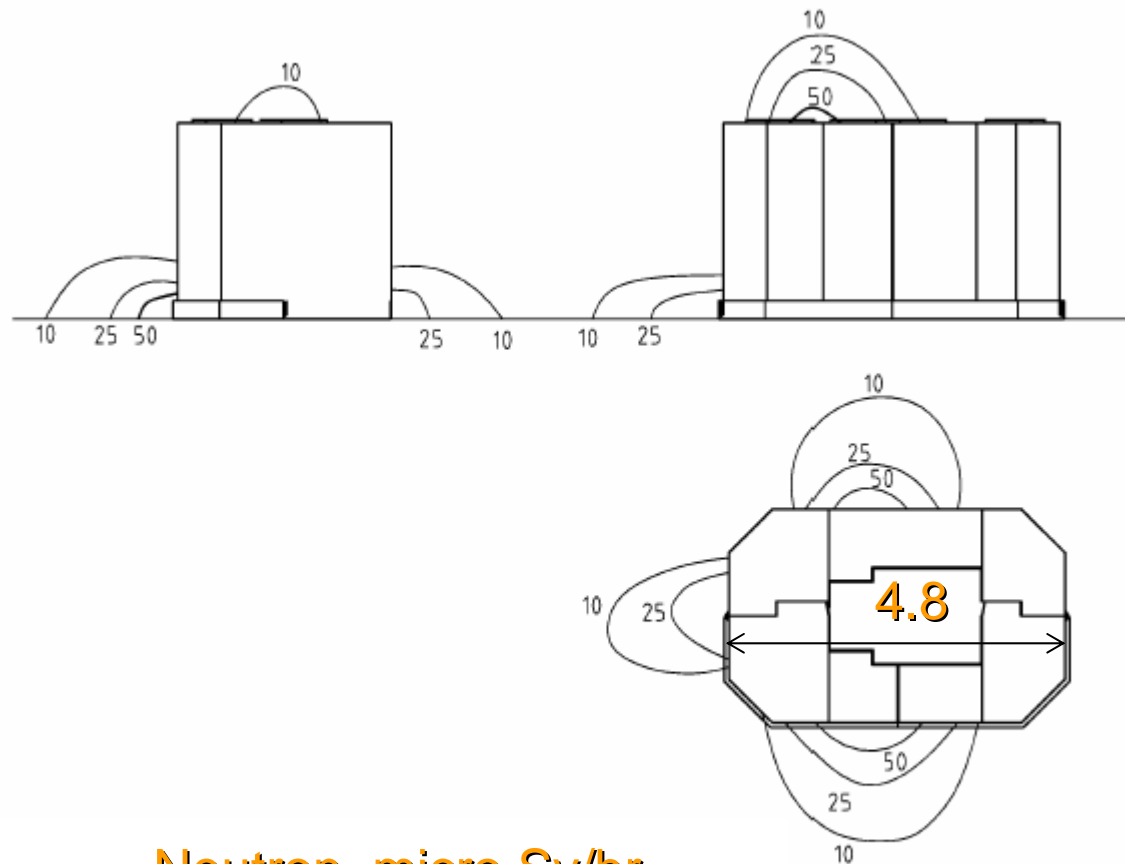
PETtrace isodose curves



Gamma micro Sv/hr
40 mA dual beam O18 T1 + T4



PETtrace isodose curves

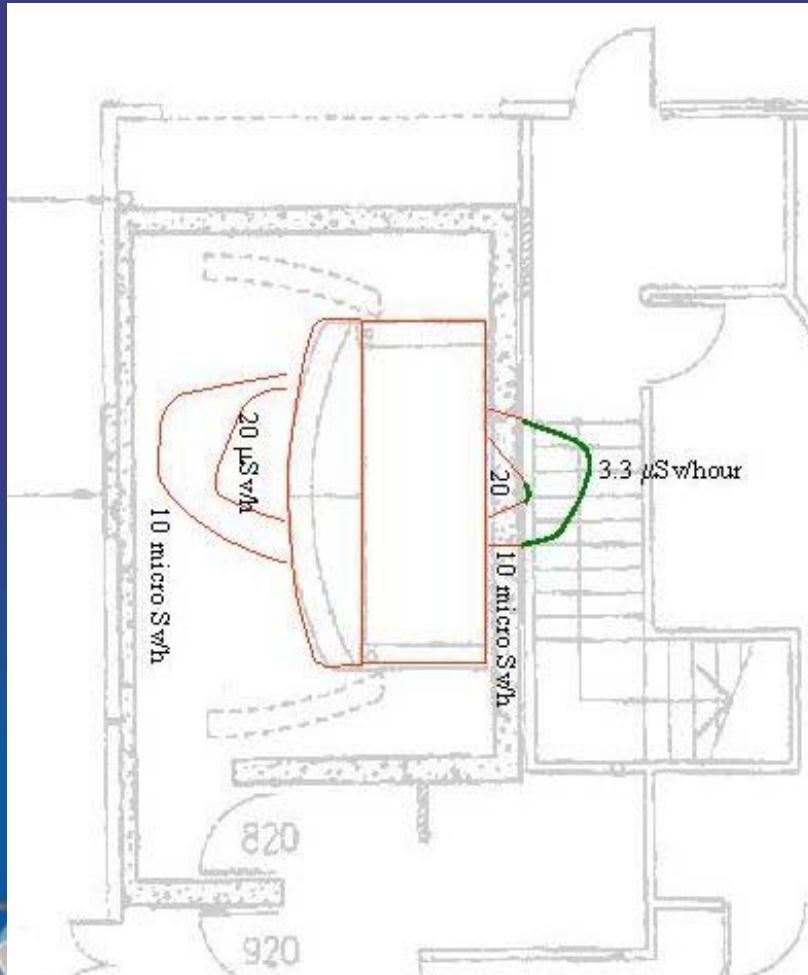


Neutron micro Sv/hr
40 mA dual beam O18 T1

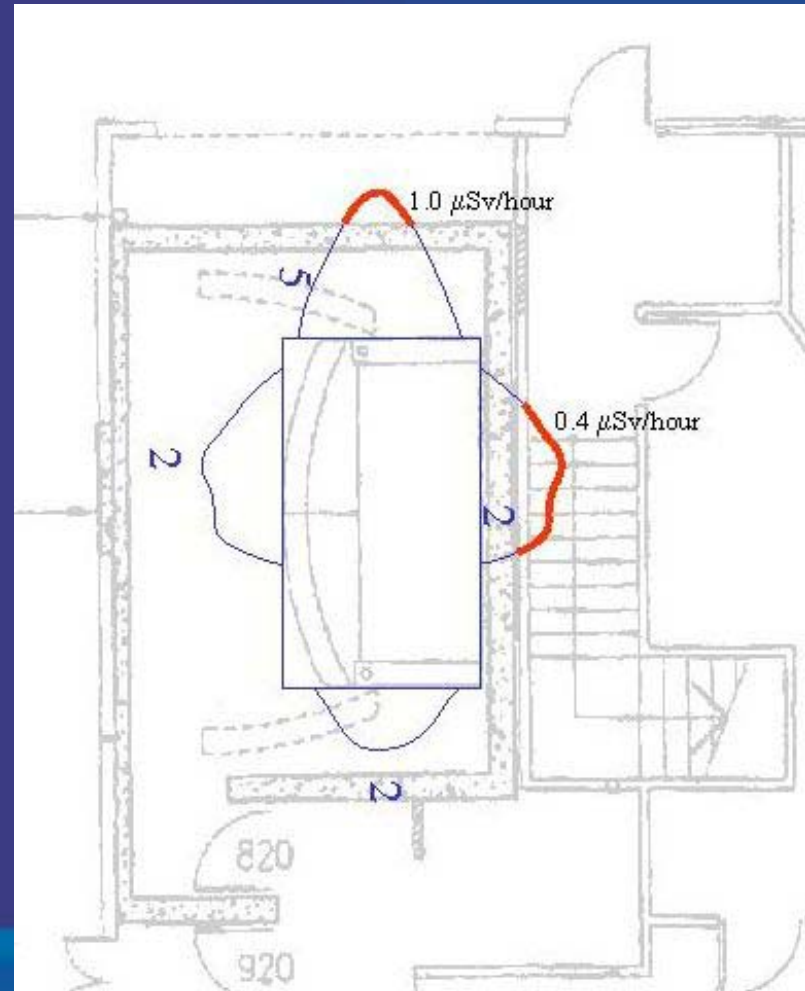


Minitrace placed with in an existing building with additional 300 mm precast concrete walls

Gamma



Neutron



Radiation Attenuation and Vaults

- Prompt Gammas are easy.
Find source terms and effective energy and look up the attenuation values in references.
- Neutrons are not easy.
The nature of, and likelihood of interaction depends on energy and material. Cross section for scatter and capture are energy dependent.



Neutrons

- Fast neutrons are elastically scattered losing energy. They do not get absorbed much. H rich compounds are good moderators.
- Thermal neutrons have lost most of their energy and are elastically scattered until they are absorbed.
- Thermal neutrons in air are often described as a neutron gas and to a degree will diffuse like a gas.
- Elements with large capture cross sections act as neutrons sinks.



Neutron capture

- Generally a (n,γ) reaction.

The γ rays are not trivial.

Boron approx. 0.3 MeV

Hydrogen 2 MeV

Iron approx. 7 MeV

These become a diffuse gamma source where ever the neutron stops, for instance partly through the vault wall,

Adding to the complexity of the gamma shielding.



Neutron Attenuation / Dose Calculation

- Empirical (for some materials) requires knowledge (assumptions) of neutron flux, energy and concrete density.
- Monte Carlo simulation of cyclotron in vault.
Requires knowledge of neutron source terms, energy distribution and relative elemental composition of cyclotron and vault.

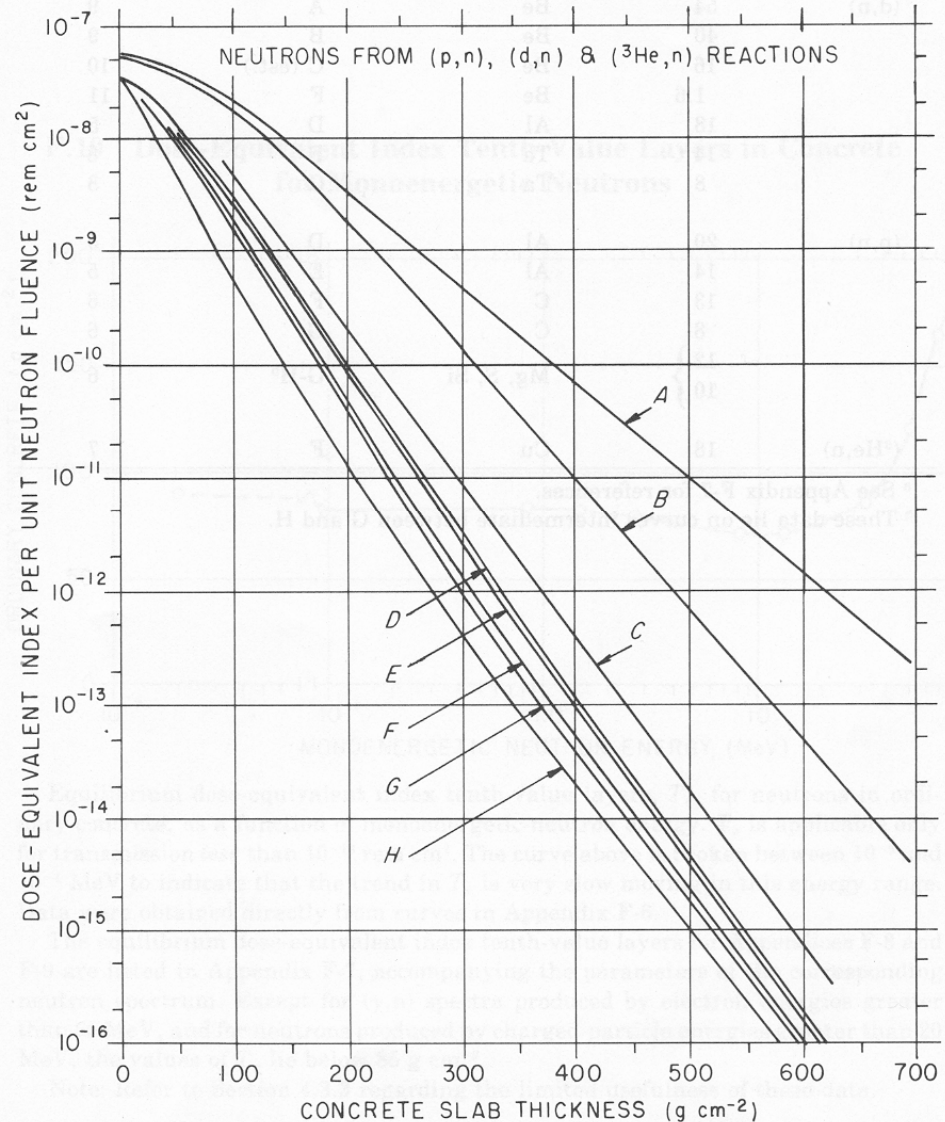


Empirical

NCRP Report No. 51 - Radiation Protection Design Guideline for 0.1 to 100 MeV: Particle Accelerator Facilities

Dose, including
gamma from (n,g)
reactions in concrete
rem cm² per unit
neutron fluence

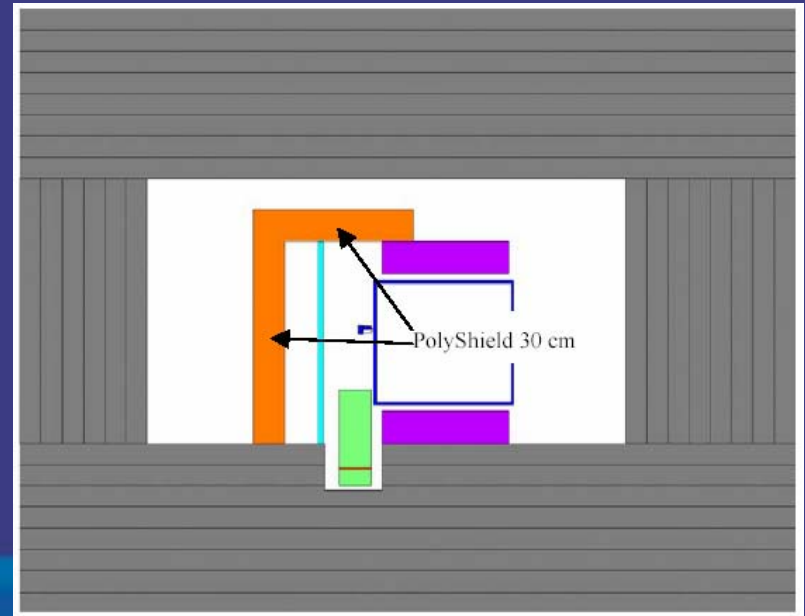
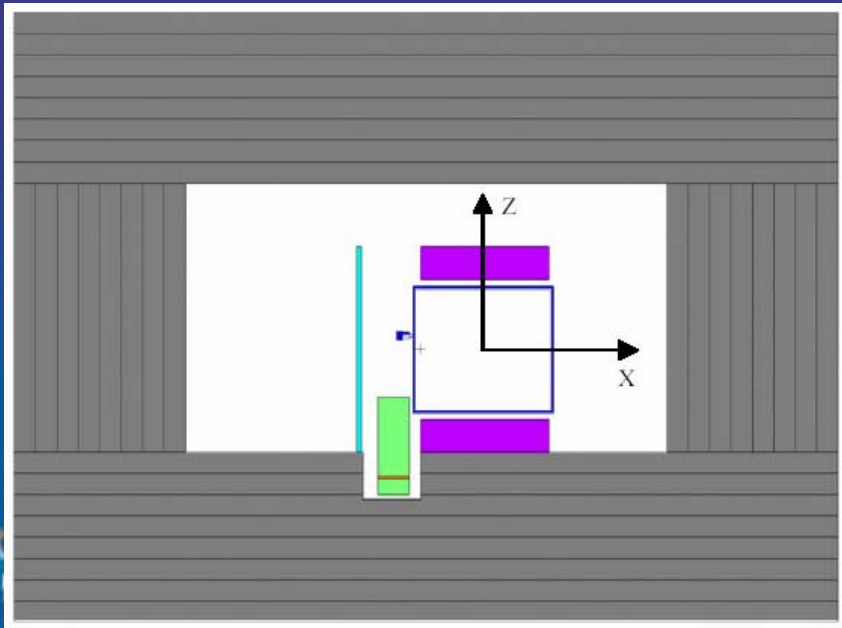
F.9 Dose-Equivalent Index Transmission Through Concrete
of Neutrons from Ion-Induced Reactions



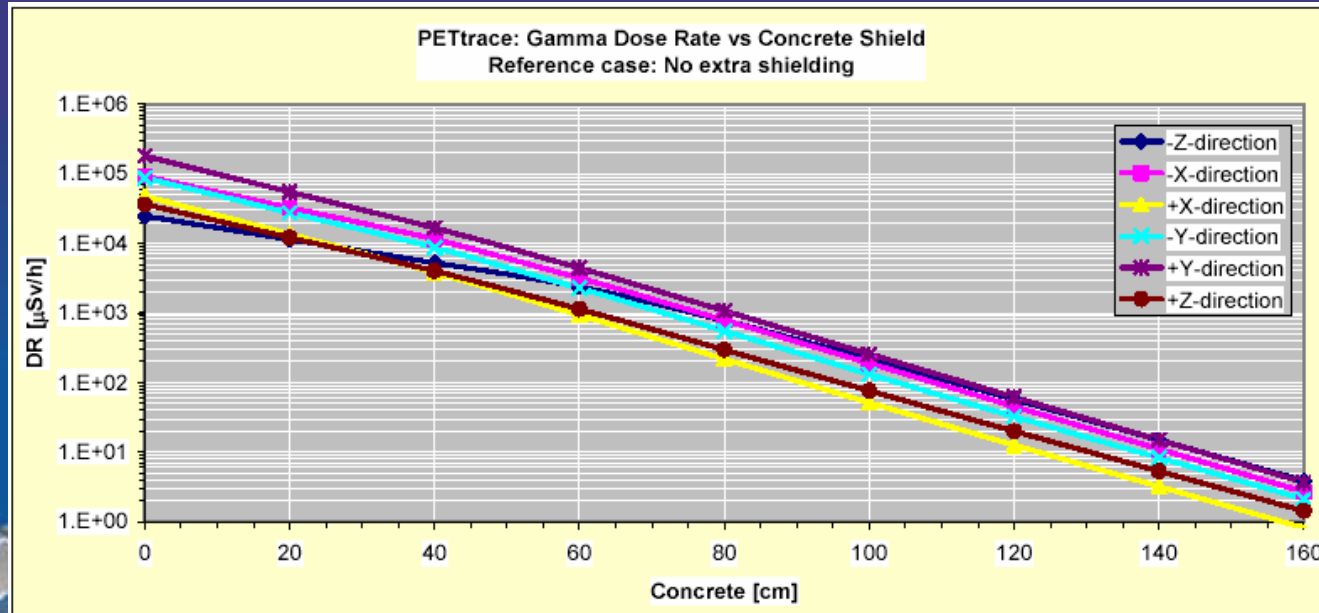
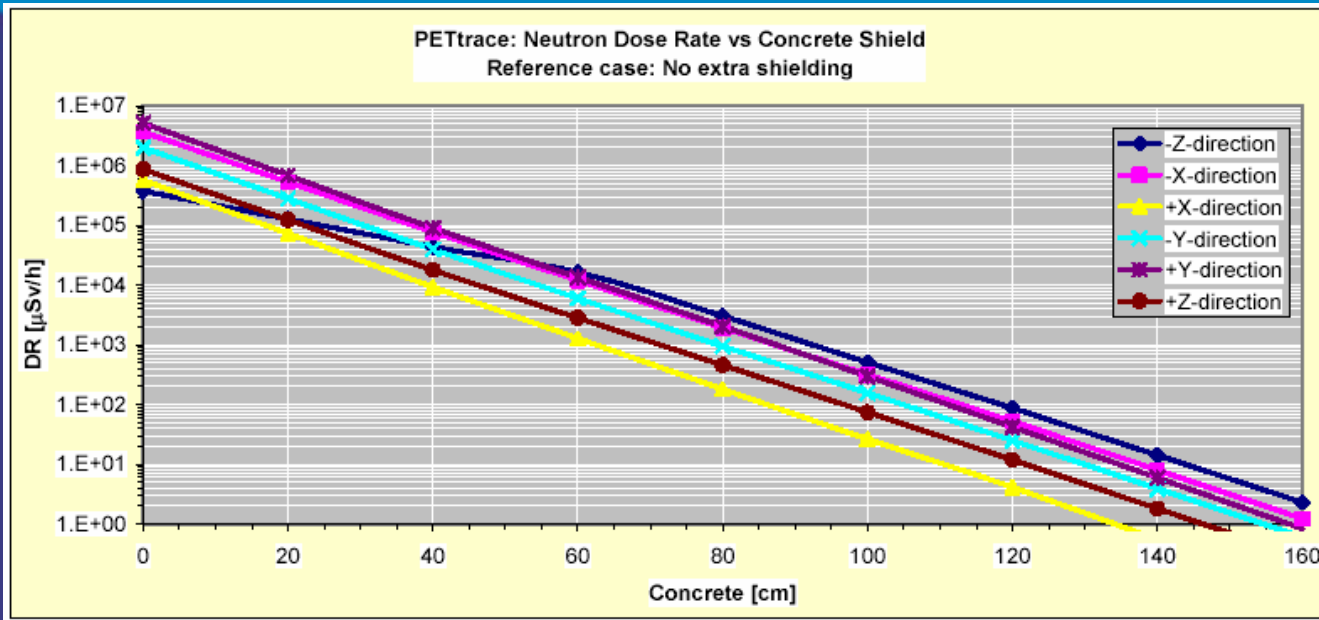
Monte Carlo

- Source neutron terms, energy and direction.
- Geometry of vault.
- Elemental composition of surrounding material eg cyclotron steel concrete target material etc
- Cross section at various energies for elements
- Run simulation to see the evolution of gamma and neutron dose.

GE PET trace simulation dose in 20 cm slabs for typical F18 run.



Monte Carlo



H_2^{18}O target
40 μA , 15 MeV
protons
(GE PET trace)

Monte Carlo

- Your vault will not be the same dimensions as the published simulation
- It is too hard to re do it your self
- The composition of the concrete will not be the same.



Shielded or vault:

Get as much information as possible

Use a Semi Empirical approach.

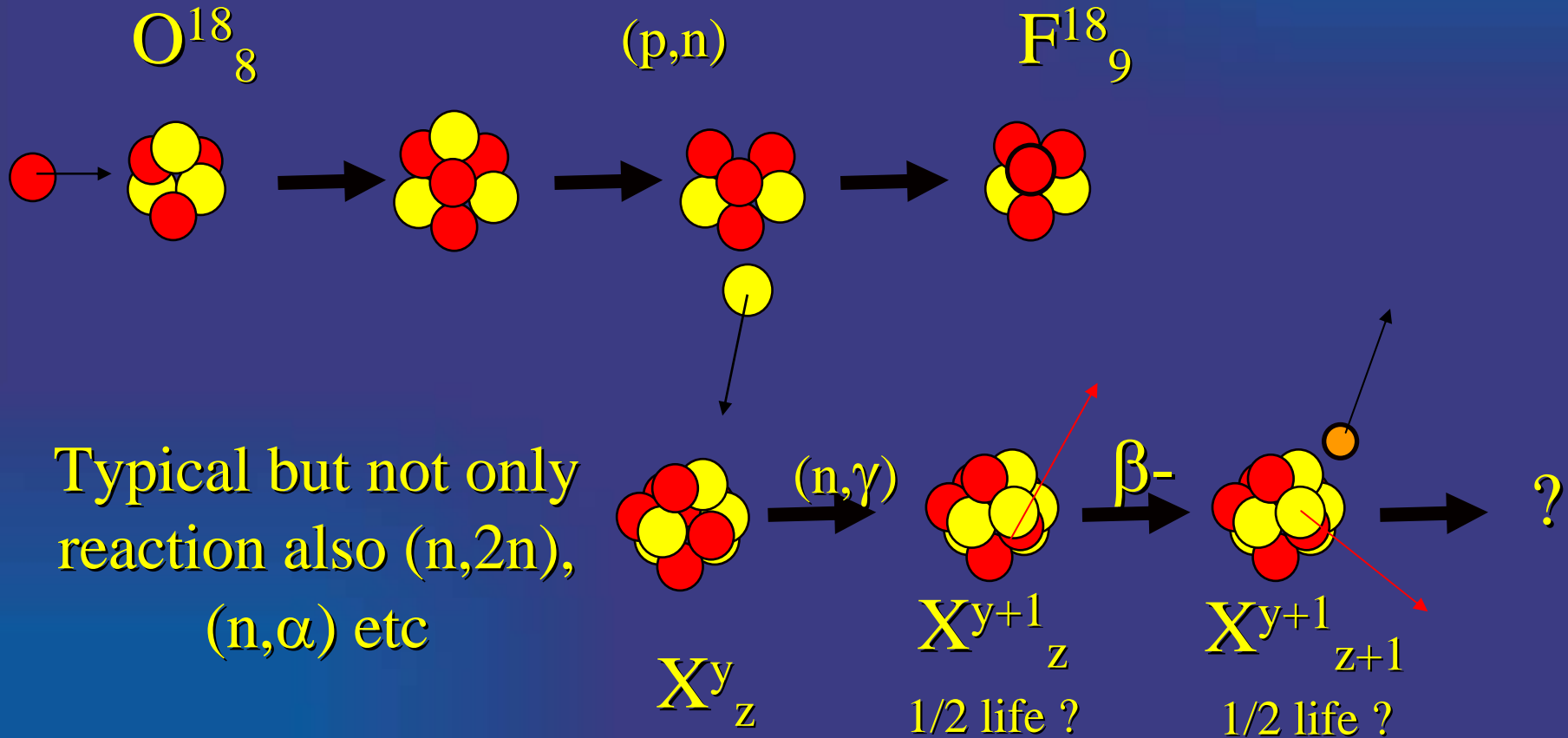
- 1) Apply judicial fudge factors to the data you have, then add at least an extra 1/2 value.
- 2) Make sure at least two approaches produce a shielding value with in the same order of magnitude.
- 3) Check with other installations



Consequences of stopping all those neutrons?

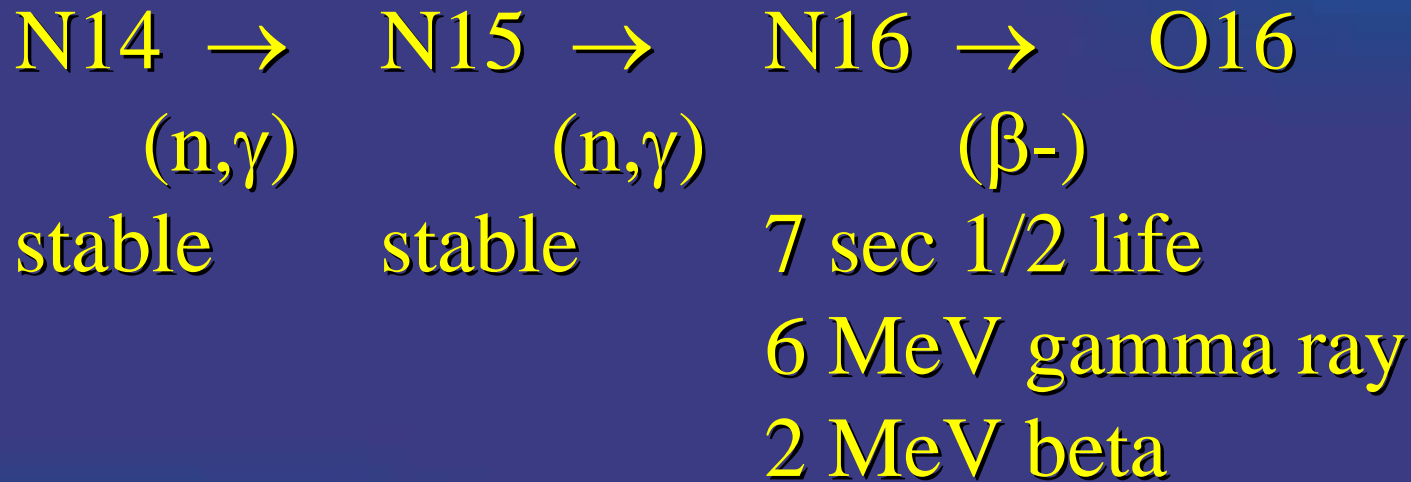
Activation.

After capturing a neutron the atom may not be stable



Activation occurs every where

Air



Negative pressure around cyclotron
slow passage up stack.



Activation after production

Induced activity in the cyclotron

A cocktail of radioactive isotopes will be formed in the cyclotron with a range of cross-sections and half-lives.

After a weekend most of low energy gamma radiation has decayed.

After isotope production, wait for at least two hours before entering the cyclotron room or opening shields. mSv per hour

Maintenance: a radiation measurement to see if work is appropriate. Plan the work. Prepare tools if needed. Stop and re assess if not going to plan.



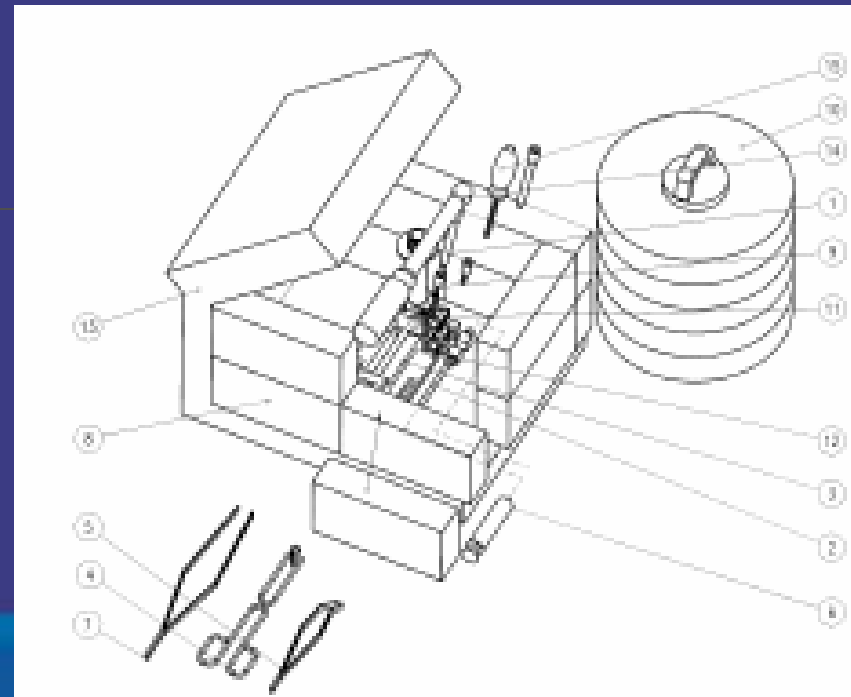
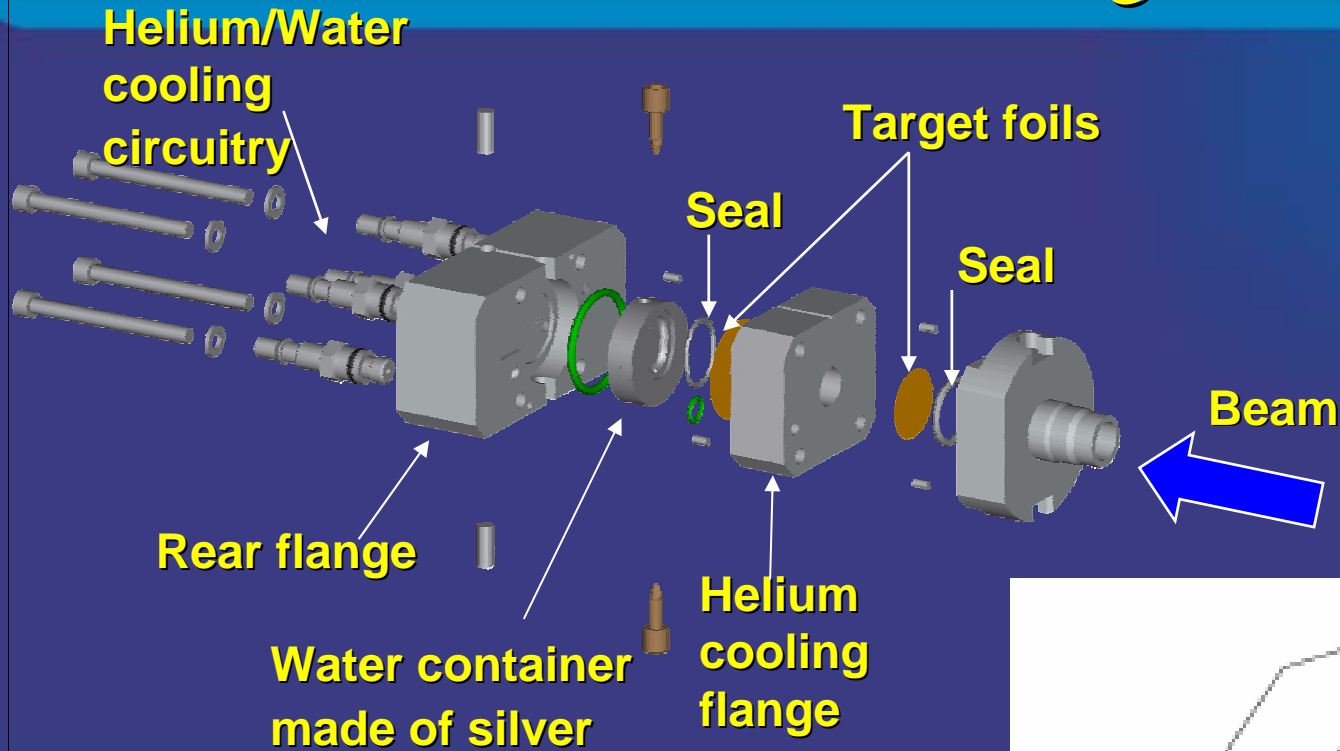
Maintenance

Protective measures

- Electronic dosimeter at all times
- Gloves
- Lab coat
- Plastic safety glasses, protection from β 's
- Use of shielding where appropriate
- Long tweezers and tongs
- Treat all components as hot, waste store.
- Do not mill or sand materials



Target Maintenance



Long Term Activation: Protons

- Most cyclotron components are not exposed to proton beam.
- Notable exception are the Havar target foils.
Co 42.5%, Cr 20%, Mn 1.6%, Mo 2%, Ni 13%,
W 2.8%, Fe 18.1%
- and the Collimators



Long Term Activation: Protons

Isotope	Half-life	Activity [Bq]	Main components
Al26	717 ky	1.3E+03	Silica, Alumina
V49	330 d	1.4E+08	Havar foil
Cr51	27.7 d	3.8E+07	Havar foil
Mn52	5.51 d	2.2E+08	Havar foil
Mn53	3.74 My	4.5E+03	Havar foil
Mn54	312.3 d	2.5E+09	Havar foil
Fe55	2.73 y	7.9E+07	Havar foil
Co56	77.27 d	7.2E+09	Havar foil
Co57	271.74 d	2.5E+08	Havar foil
Co58	70.86 d	1.8E+08	Havar foil
Ni57	6.077 d	1.4E+06	Havar foil
Ni59	76 ky	4.4E+06	Havar foil
Ni63	100.1 y	7.6E+04	Havar foil
Zn65	244.26 d	3.4E+08	Copper
Ta179	1.82 y	4.7E+05	Ta Collimator
W181	121.2 d	1.7E+10	Ta Collimator

20 years of operation, 20% utilisation followed by a 30 day decay period



Long Term Activation: Neutrons

- Most cyclotron components are selected to have low activation profiles and or short 1/2 life products.

Isotope	Half-life	Activity [Bq]	Main components
Cr51	27.7 d	2.5E+07	Aux. Equipment
Mn54	312.3 d	4.8E+07	Acc. Cell, Iron yoke
Fe55	2.73 y	5.4E+08	Acc. Cell, Iron yoke
Fe59	45.1 d	1.3E+07	Acc. Cell, Iron yoke
Co58	70.86 d	1.1E+07	Aux. Equipment
Co60	5.27 y	9.8E+07	Aux. Equip., Iron yoke
Ni59	76 ky	1.6E+04	Aux. Equipment
Ni63	100.1 y	3.7E+06	Aux. Equipment
Ag108m	418 y	7.9E+05	Target holder (Ag)
Ag110m	249.9 d	7.2E+07	Target holder (Ag)

Un-shielded PET trace

20 years of operation, 20% utilisation followed by a 30 day decay period



Long Term Activation: Neutrons

Shielded PETtrace

Isotope	Half-life	Activity [Bq]	Main components
H3	12.33 y	7.8E+07	Concrete floor
Be10	1.6 My	8.0E+01	Polyethylene (3% B)
C14	5730 y	1.5E+03	Concrete floor
Ca41	103 ky	2.0E+04	Concrete floor
Cr51	27.7 d	2.0E+07	Aux. Equipment
Mn54	312.3 d	3.7E+07	Acc. Cell, Iron yoke
Fe55	2.73 y	4.3E+08	Acc. Cell, Iron yoke
Fe59	45.1 d	1.0E+07	Acc. Cell, Iron yoke
Co58	70.86 d	8.8E+06	Aux. Equipment
Co60	5.27 y	8.0E+07	Aux. Equip., Iron yoke
Ni59	76 ky	1.3E+04	Aux. Equipment
Ni63	100.1 y	2.9E+06	Aux. Equipment
Ag108m	418 y	6.1E+05	Target holder (Ag)
Ag110m	249.9 d	5.6E+07	Target holder (Ag)
Eu152	13.32 y	2.3E+06	Concrete floor
Eu154	8.6 y	2.1E+05	Concrete floor

20 years of operation, 20% utilisation followed by a 30 day decay period



Long Term Activation: Neutrons

Un Shielded PETtrace Vault Walls

Nuclide	Specific activity [Bq/kg]				Total [Bq]
	0-20cm	20-40cm	40-60cm	140-160cm	
Co60	3.1E+03	1.0E+03	2.3E+02		2.2E+08
Cs134	5.0E+02	1.1E+02	2.2E+01		3.2E+07
Eu152	4.2E+03	1.5E+03	3.6E+02		3.2E+08
Eu154	2.8E+02	8.0E+01	1.7E+01		1.9E+07
Fe55	1.2E+04	4.4E+03	1.0E+03		9.0E+08
Fe59	2.0E+02	7.2E+01	1.6E+01		1.5E+07
H3	6.3E+04	2.3E+04	5.2E+03		4.7E+09
K40	6.2E+02	6.2E+02	6.2E+02		3.9E+08
Th232	3.2E+01	3.2E+01	3.2E+01		2.0E+07
U238	2.5E+01	2.5E+01	2.5E+01		1.5E+07
Total	8.5E+04	3.1E+04	7.6E+03		6.7E+09

20 years of operation, 20% utilisation followed by a 30 day decay period



Planning for decommissioning at installation

- Long term activation in cyclotron
 - (organise storage in advance)
- Long term activation of self shield
 - (organise storage in advance)
- Long term activation of floor
 - Limestone mix in sacrificial topping over slab
- Vault walls / ceiling
 - Limestone mix in sacrificial topping over slab
 - and or neutron moderators absorbers around targets



Production

- Products
 - small volumes of material
 - well shielded in trenches / ducts running to hot cells
 - typically 50 mm of lead
- Chemistry in Hot cells





Production

- 2/3 of cyclotron problems are due to blockages, leaks, faulty valves, clogged filters and material getting where it shouldn't.



Radiation Monitoring

Continuous monitoring + logging

- Gamma in vault or near self shield
 - Gamma outside vault
 - Stack monitoring
 - Air concentration
 - Gamma in hot lab.
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- Periodic Gamma + Neutron surveys

